

# **The Chinese Magnetic Fusion Program**

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**On behalf of MCF community of China**

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# MCF research largely guided by National Magnetic Confinement Fusion Science Program ( MOST )

- ❑ To support R&D needed for ITER construction ( PAs ) and Operation
- ❑ To establish S&T basis required for CFETR design and construction

**In last few years, projects distribute on:**

- **Research of Integrated CFETR design**
  - For proposing project, figuring-out R&D needs
- **Fusion Science research in support of ITER and CFETR**
  - Augmenting research capabilities ( EAST and HL-2M; facilities in uni.-level)
  - Research on reactor-relevant key physics and technology (TC, MHD, EP, Dia, Contrl...)
  - Research on development of ITER/CFETR relevant operation scenarios.
  - Extending collaboration based on EAST/HL-2M (international, domestic , ITPA)
- **R&D of Reactor key technology and components** (in addition to ITER PAs)
  - Advanced superconducting magnets and vacuum vessel
  - Heating and current drive technologies
  - Fusion reactor material in component scale
  - Tritium self-sufficient technology and Remote handling
- **University Research and Young Scientist program**



# Physics Projects in National Magnetic Confinement Fusion Science Program support Fusion Community diversely

## ● EAST on integrated advanced steady-state scenarios

- SSO scenarios (high  $f_{bs}$ ,  $\beta_p$ ,  $f_{GW}$ ; ELM and impurity control, core-edge integration...)
- ITER Pre-Nuclear phase scenarios (H/He plasma and full-metal wall)
- Collaboration (ITPA:Div/SOL、PEP, international and domestic: high  $\beta_p$ , H/He plasma )

## ● HL-2A/2M on high performance scenarios

- High performance operation scenarios (high  $\beta_N$ ,  $I_p$ ; Div/SOL and MHD control, EP...)
- Collaboration (ITPA:MHD、TC, international and domestic: Edge、AI Control)

**Mission-driven  
EAST&HL-2A/M  
joint actions  
User facilities**

## ● University and Non-tokamak researches

- University scale facilities: JTEXT, FRC, Stellerator, ST, linear-device for PMI...
- Theory/modeling/simulation and diagnostics (EP, MHD, TC,...)

## ● Young Scientist program (majorly oriented to Uni.)

- Free research (two topics with 5 persons/each/year)
- Encourage collaboration with major facilities

**Free-research  
Mission-driven  
Supporting team  
Education/Training**



# Major fusion device in China



**EAST**



**HL-2A**



**HL-2M**



**Congratulation to 1000s!**

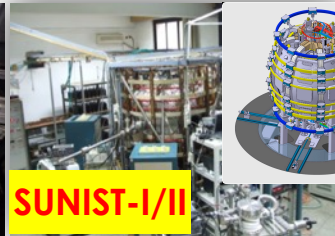
**Congratulation to 1MA!**



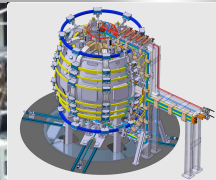
CFQS 主机



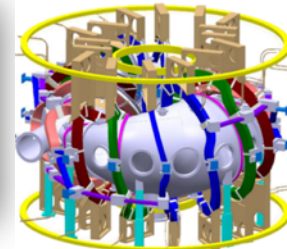
**J-TEXT**



**SUNIST-I/II**



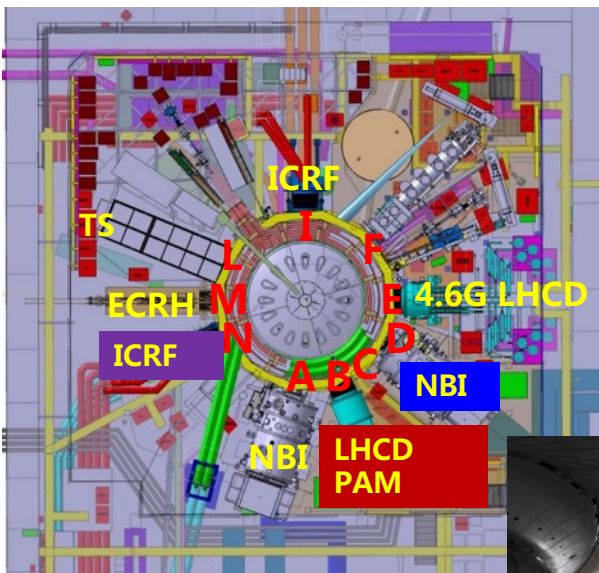
**KTX**



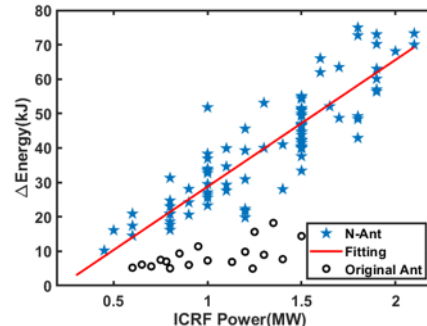
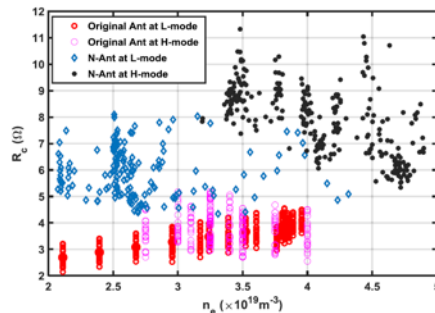


# **Major Progress on EAST and HL-2A/M Experiments**

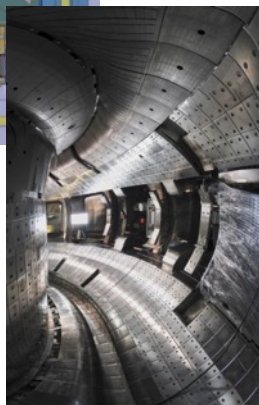
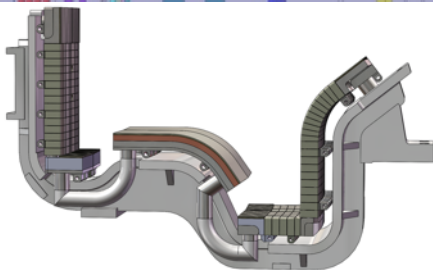
# Recent EAST Upgrade allow higher H&CD power & capable of exploring scenarios with full metal wall



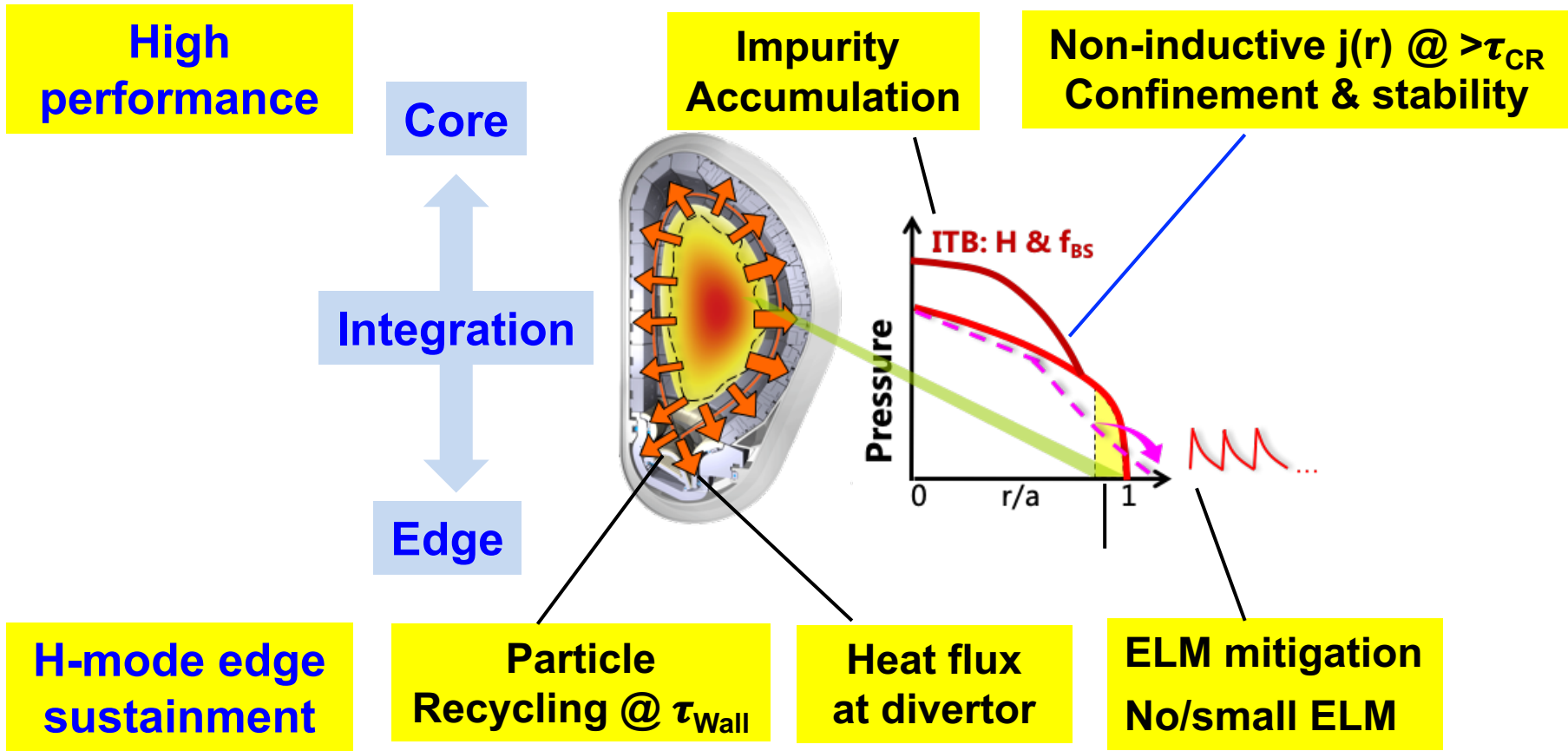
- New ICRF antenna with smaller  $k_{||}$  improves coupling and heating efficiency



- One more gyrotron (1MW@140GHz)
- New lower W-divertor: closed outer- and open inner divertors for balanced detachment
- Since 2020 full metal PFC



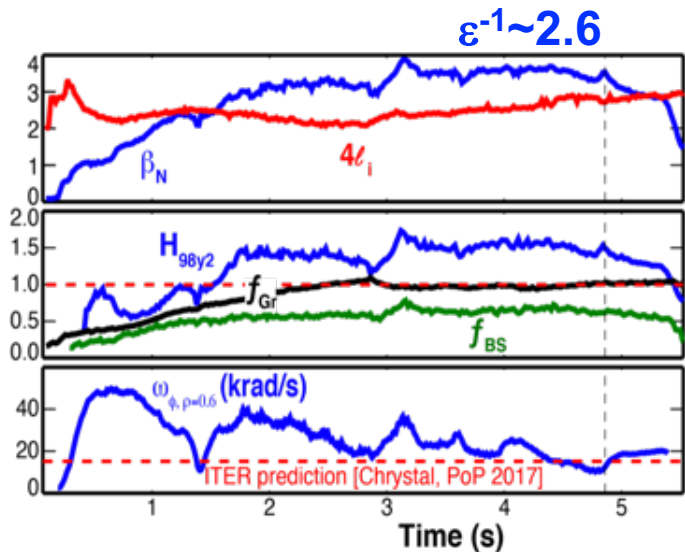
# Efforts on Key issues in supporting long pulse H-mode operation on EAST





# Joint DIII-D/EAST research on high $\beta_p$ H-mode scenario

## Broad current profile delivers high stability & confinement



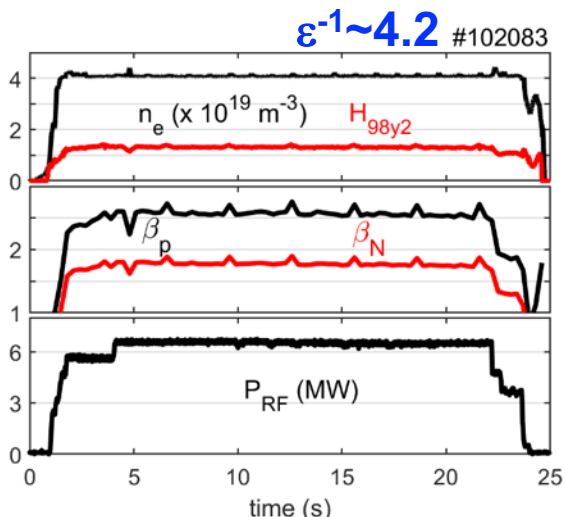
*J. Huang, NF (2020), A.M. Garofalo, NF(2015)*

$$f_{BS} \sim \beta_P \sqrt{\epsilon}$$

High  $\beta_N$

Long pulse

High  $H_{98y2}$ ,  $n_e$ , &  $f_{BS}$   
Low rotation



*X. Gong, NF (2019)*

DIII-D fully non-inductive

- Balanced beam, low torque
- $H_{98y2} \sim 1.5$ ,  $\beta_P \geq 3$ ,  $\beta_N \geq 4 * I_i$ ,  $f_{bs} \sim 0.8$

EAST fully non-inductive, **long pulse**

- RF:  $H_{98y2} \sim 1.3$ ,  $\beta_p \sim 2.5$ ,  $\beta_N \sim 1.8$ ,  $f_{bs} \sim 0.5$
- RF + NB:  $H_{98y2} \sim 1.2$ ,  $\beta_p \sim 3.1$ ,  $\beta_N \sim 2.1$

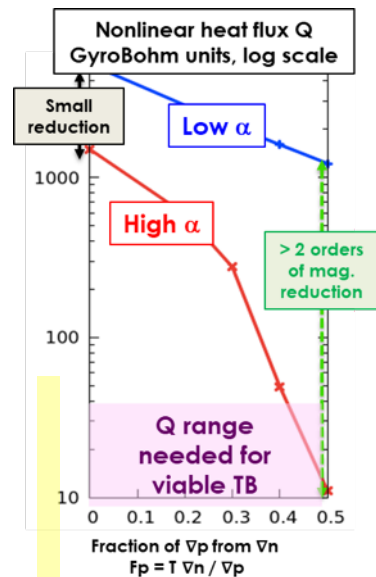
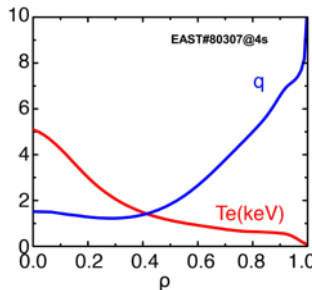
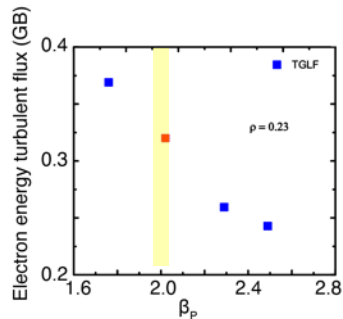
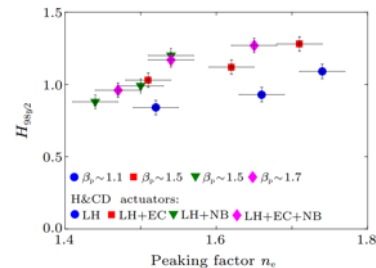
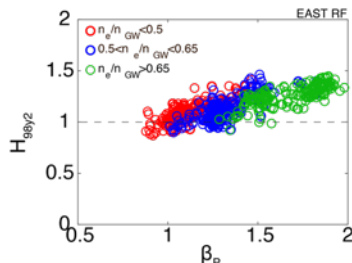




# The physics mechanism behind the improved energy confinement in long-pulse high $\beta_p$ H-mode on EAST

- **Improved  $H_{98y2}$  at higher  $\beta_p$** 
  - TGLF: turbulent  $Q_e$  decreases with the increase of  $\beta_p$
  - High  $\beta_p$  facilitates weak negative shear, and promote ITB formation
- **$H_{98y2}$  depends on  $\nabla n_e$  in various Heating/CD mixtures**
  - TGLF: enhanced Shafranov shift stabilizing effect on turbulent transport by high  $\nabla n_e$  in high  $\beta_p$  regime
  - Instability cannot feed  $\nabla n_e$  due to poor coupling to trapped electrons

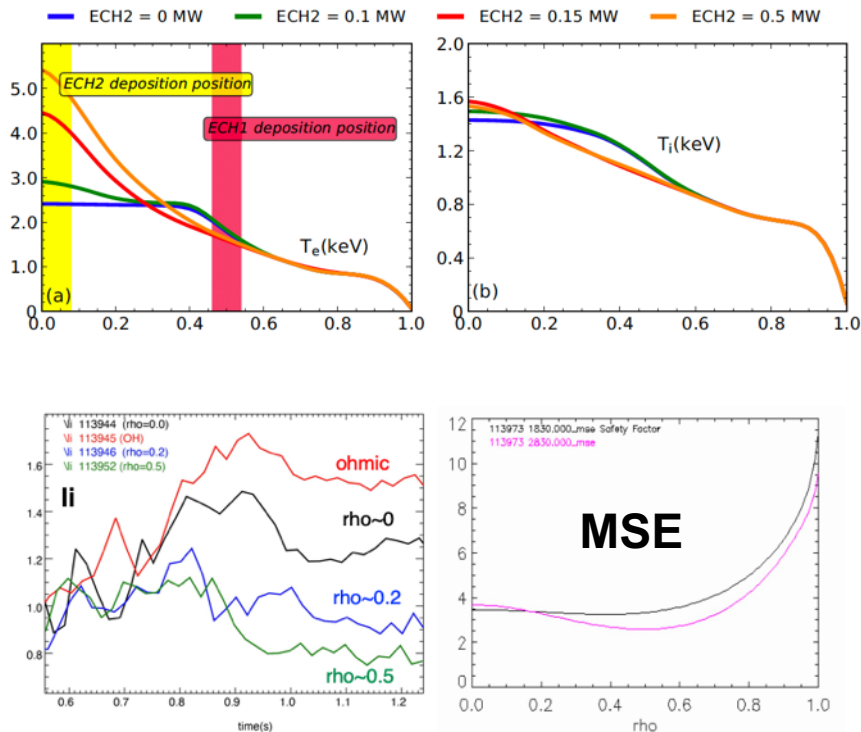
*J.P. Qian, PoP (2021)*  
*B.N. Wan, CPL (2020)*





# Modeling and experiment show that current profile becomes broader with off-axis EC

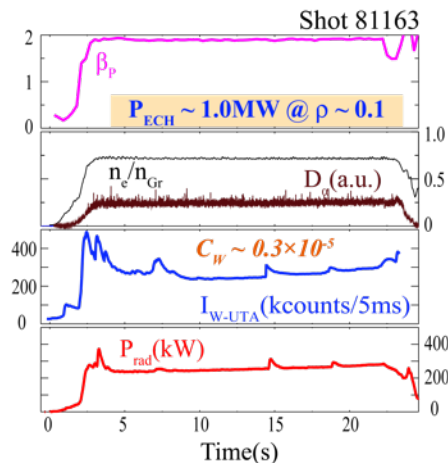
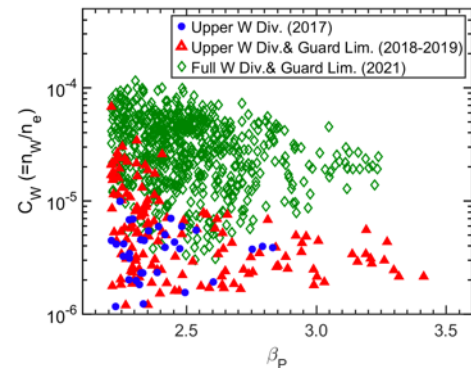
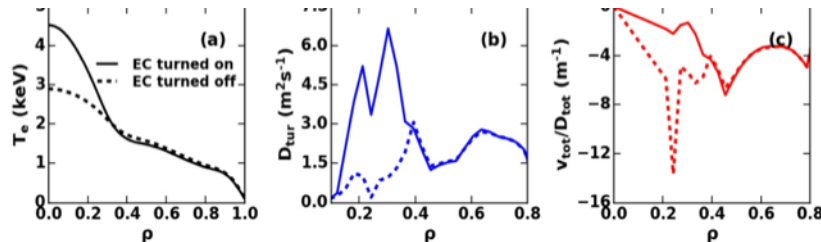
- ITB inside  $\rho < 0.4$ , could we expand?
- Centrally deposited ECH increases  $j(r)$  and  $T_e$  on the axis
- Early heating of off-axis EC resulted in reversed shear with lower  $l_i$ 
  - Challenge for low current ramping-rate and high aspect ratio
  - $q_{\min} > 2.0$  observed with MSE constraint
- Exploring for higher  $\beta_P$  &  $f_{BS}$  with ITB at larger radius, but challenge for steady-state





# Low $C_W$ is maintained with full tungsten divertor operation

- $\sim 4$  times increase of  $C_W$  with full tungsten divertor
- Small ELMs and high density reduced W-sputtering
- W accumulation in the improved core confinement regime due to large **neoclassical inward convection**
- Tungsten accumulation avoided by on-axis ECH (QuaLiKiz and NEO Modeling)
  - Increased turbulent diffusion of TEM inside  $\rho < 0.45$
  - Reduced the ratio  $V_{pin}/D_{tot}$

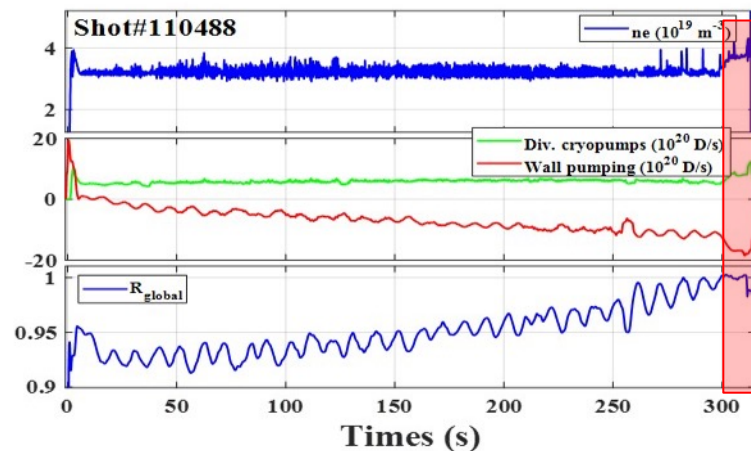
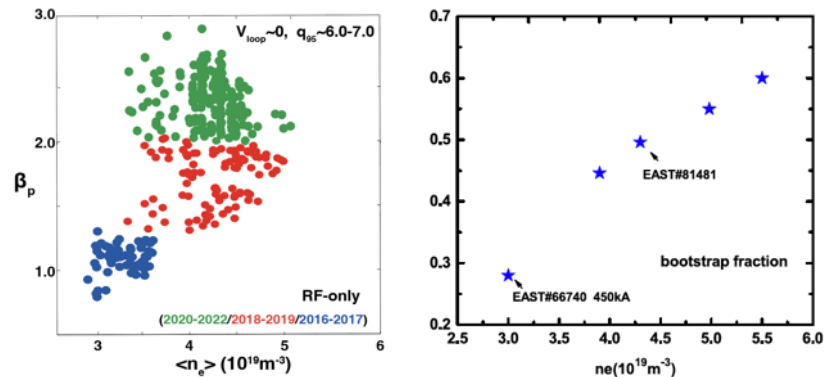


- Similar effect also observed by on-axis ICRH

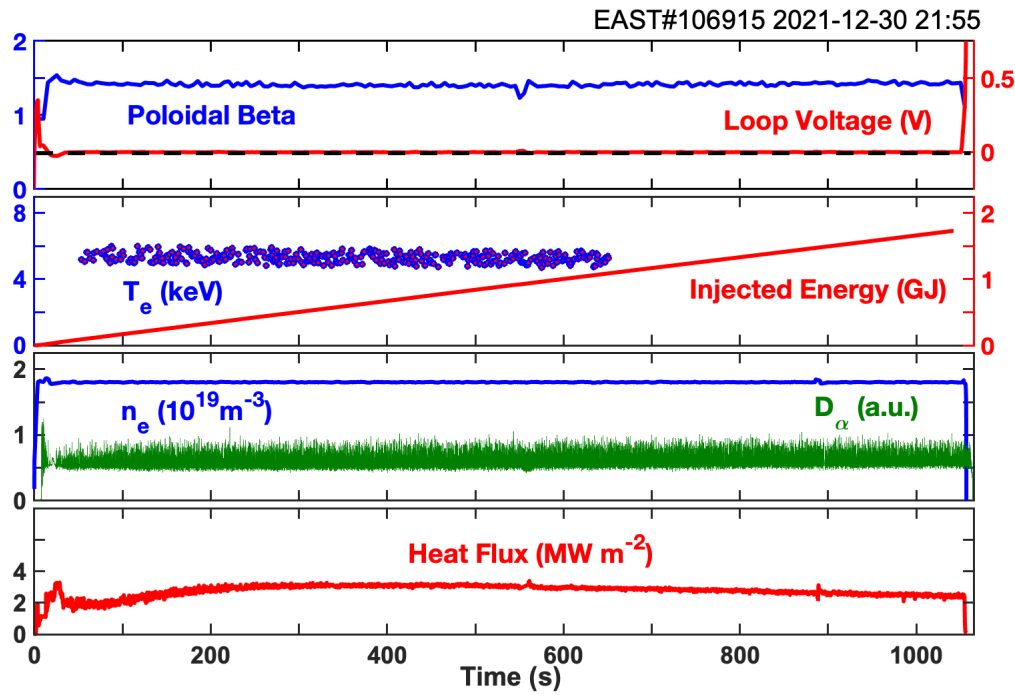


# The regime of high $\beta_p$ scenario applied for long pulse H-mode operation under ITER-like conditions

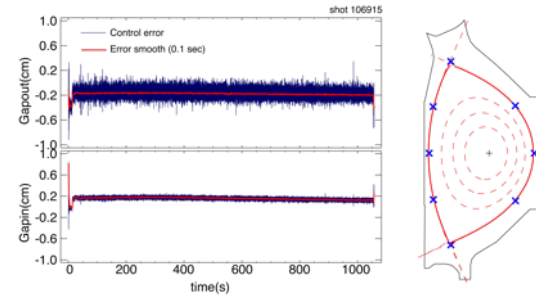
- Fully non-inductive with  $q_{95} \sim 6.0-7.0$
- Extended operational space with improved conf.
  - High  $f_{GW} \sim 0.8$ ,  $\beta_p \sim 3$ ,  $\beta_N \geq 2$ ,  $f_{bs} \geq 50\%$
  - $H_{98y2} \sim 1.4$  with e-ITB inside  $\rho < 0.4$
- ELM and tungsten impurity control
- ITER-like full W-divertor & e-heating dominant and low torque injection
- 300s H-mode with optimized recycling control, but eventually terminated by recycling
  - RF-only:  $P_{LHW} \sim 2\text{MW}$ ,  $P_{EC} \sim 1.6\text{MW}$ ,  $P_{IC} \sim 1.2\text{MW}$
  - $H_{98y2} > 1.3$ ,  $f_{Gr} \sim 0.6$ ,  $\beta_p \sim 2.5$ ,  $\beta_N \sim 1.6$ ,  $f_{BS} > 50\%$
  - Small ELM ( $f_{ELM} > 2.5\text{kHz}$ )



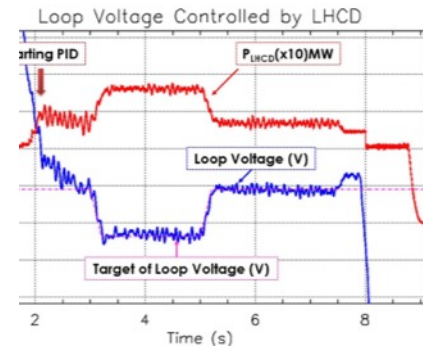
# 1056s long pulse plasma achieved with roust plasma control under full metal wall



1.7GJ energy injected and exhausted



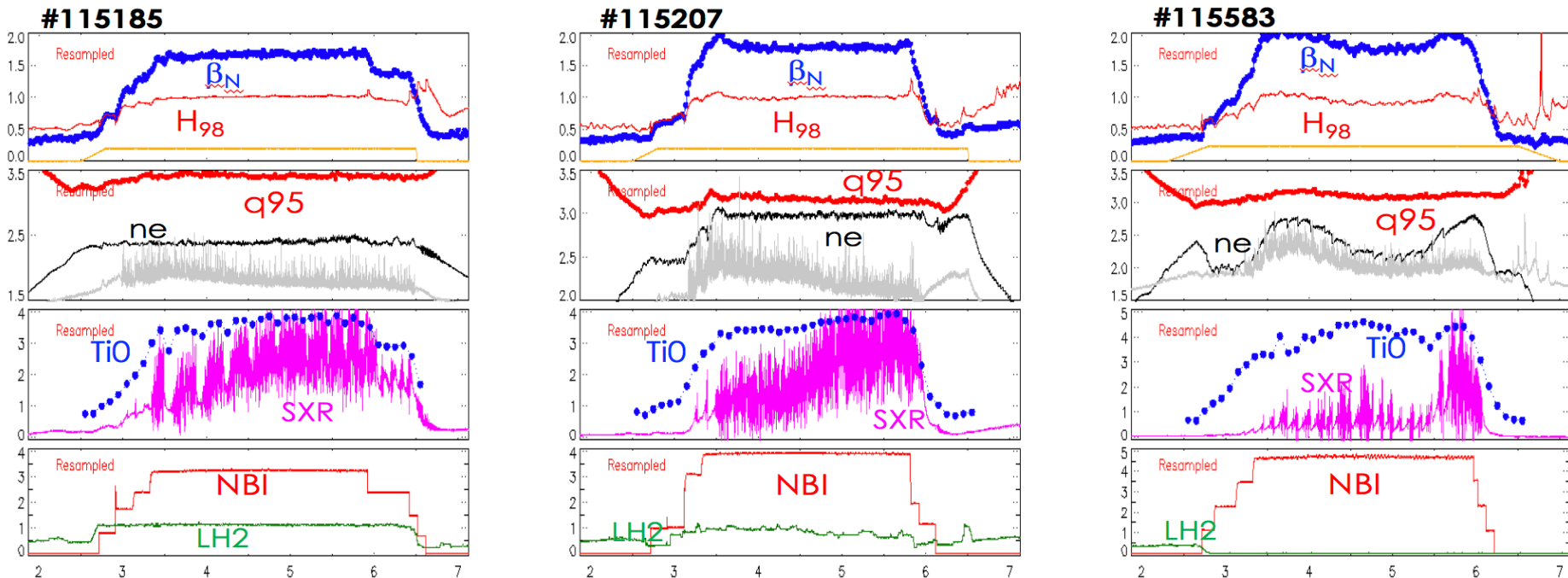
Shape control with new fiber optic current sensors, low drift integrator  
Gap < 2mm, X-point(R,Z) < 3/5mm



Loop control for fully non-inductive CD

# Develop high performance inductive scenario towards ITER baseline-like scenario

- Stationary inductive scenario at  $q_{95} \sim 3$  ( $B_T \sim 1.5T$ ) with  $\beta_N > 1.8$  and  $H_{98} \geq 1.0$
- Higher  $T_{i0}$  correlates with formation of ITB accompanied with fishbone activities
- A new 1MW gyrotron at 105GHz nearly ready for W pumping-out

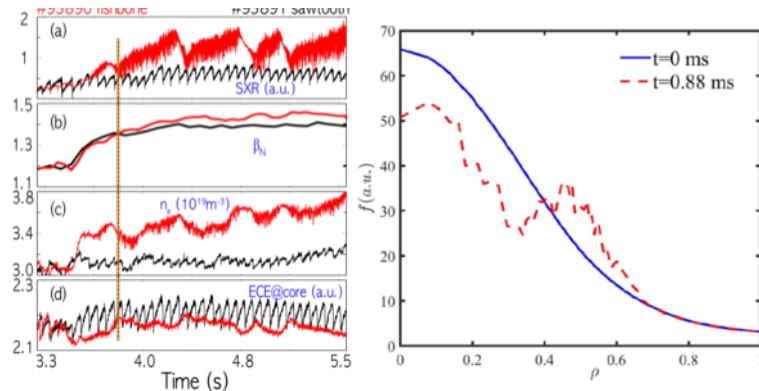




# ITB Formation correlated with Fast-ion Redistribution

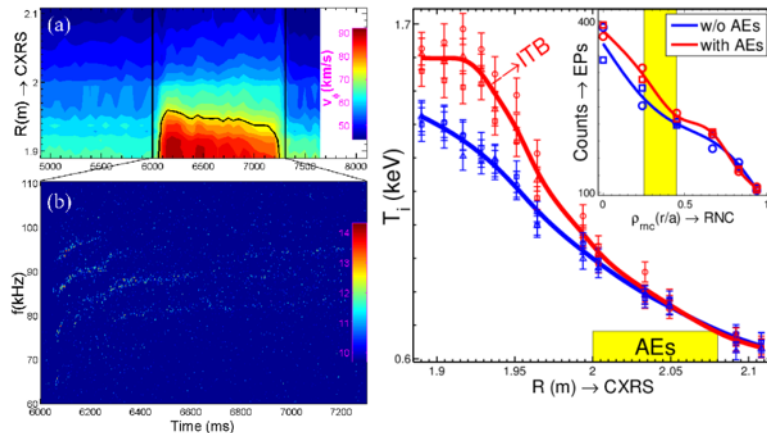
## At lower $B_T \sim 1.5T$ , $q_{95} \sim 3.5$

- Fishbone triggered at configuration with resonant  $q=1$ ;
- ITBs formed simultaneously with fishbone activities;
- M3D-K suggests the role of fast-ion redistribution.



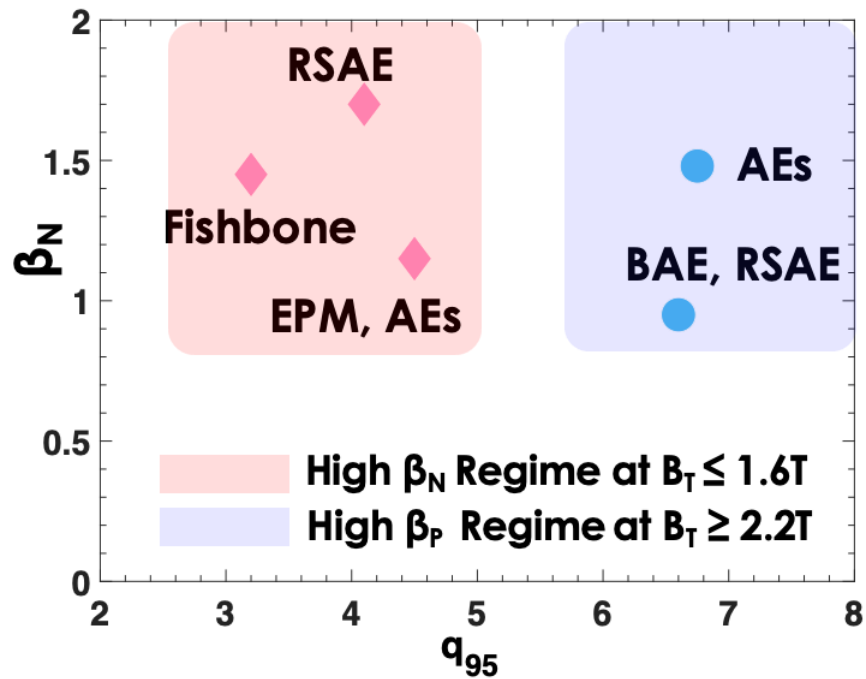
## At higher $B_T \sim 2.2T$ , $q_{95} \sim 6$

- ITBs formation accompanied with AE excitation;
- Radial neutron camera (RNC) shows redistribution of fast ions.

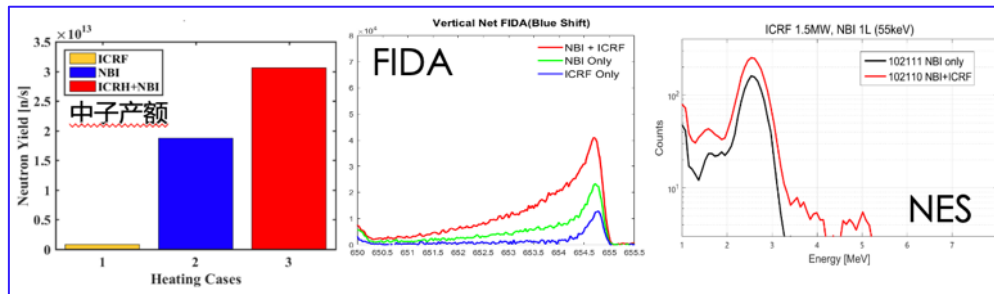




# EP related instabilities in high beta regime on EAST



- EP instabilities easier excited: Lower  $B_T$  &  $n_e$ , Reverse shear,  $q \sim 1$
- Recently enhanced ICRF ( $\sim 4\text{MW}$ ) and NBI capabilities create more EPs and accelerate EPs to MeV

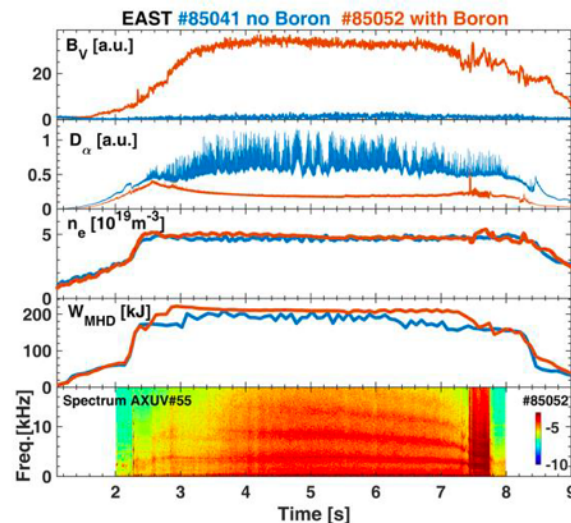
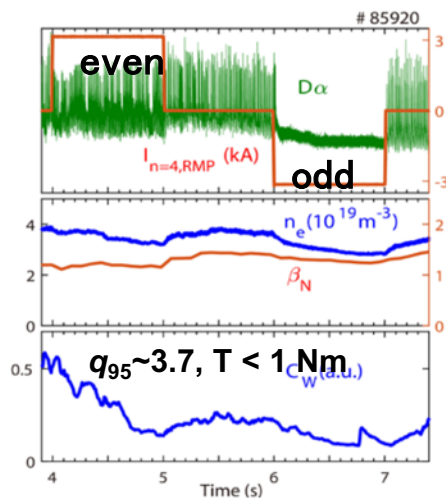
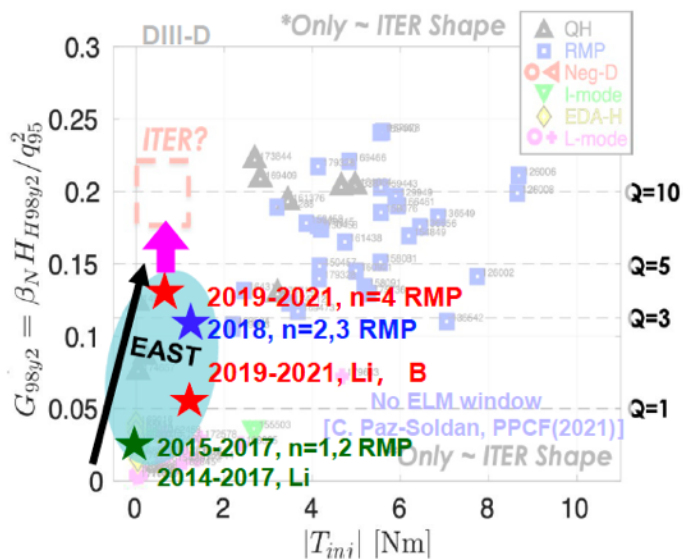




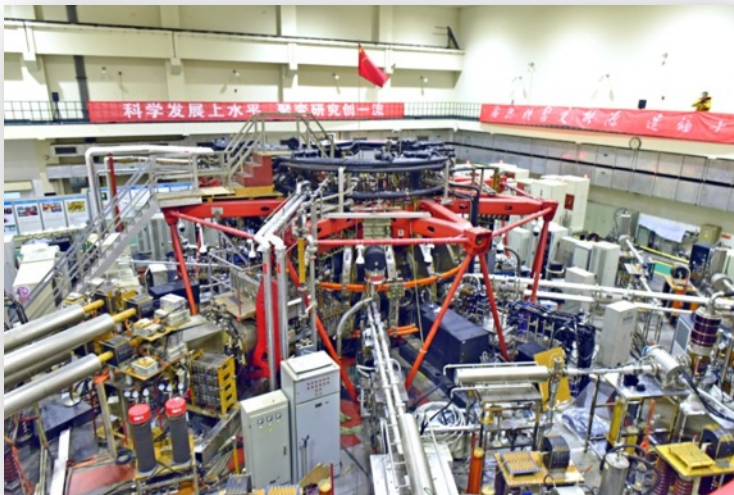


# Full ELM suppression in low input torque plasmas in support of ITER

- ELM suppression by n=4 RMP with small cost of energy confinement
- ELMs fully suppressed by Boron power injection with slightly improved confinement



# Features of HL-2M/A allow to access advanced tokamak regimes with flexible divertor configuration



- $R$ : 1.65 m
- $a$ : 0.40 m
- $B_t$ : 1.2~2.7 T
- $I_p$ : 150 ~ 480 kA
- $n_e$ :  $1.0 \sim 6.0 \times 10^{19} \text{ m}^{-3}$
- $T_e$ : 1.5 ~ 5.0 keV
- $T_i$ : 0.5 ~ 3.5 keV
- $\beta_N$ :  $>3$

- Heating systems:
- co-NBI (tangential):  
3 MW/48keV/1s
  - ECRH/ECCD:  
68G/140G, 5 MW,  
500Hz modulation
  - LHCD: 3.7G/2 MW

$I_p = 2.5$  (3) MA

$R/a = 2.8$  (1.78/0.65)

$\kappa = 1.8\text{-}2$ ;  $\delta > 0.5$

$B_T = 2.2$  (3) T

H&CD: 25 (27) MW

(NBI 15+EC 8+LH 2(4))

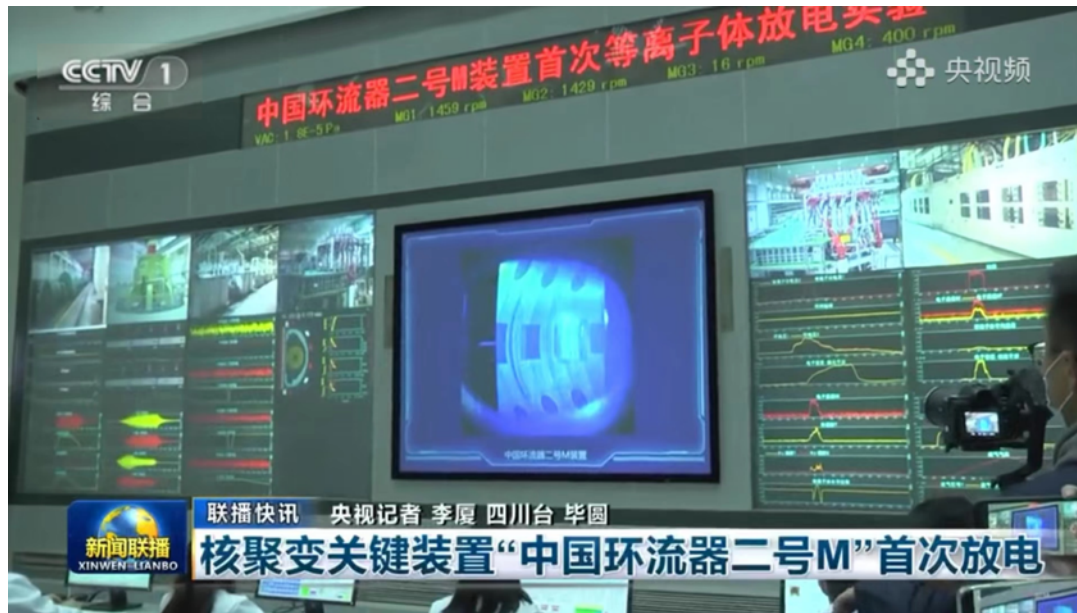
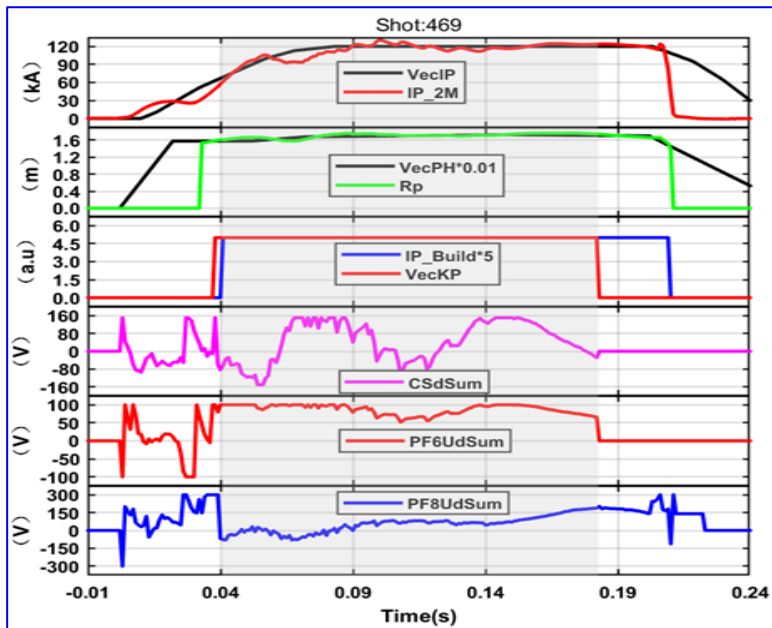
- AT: high  $f_{BS}$ ,  $\beta$ , ...
- Divertor: snowflake, super-X, ...
- Integrated core-edge solution
- MHD instabilities and disruption
- EP physics,
- ...



# HL-2M achieved first plasma on Dec. 4th, 2020

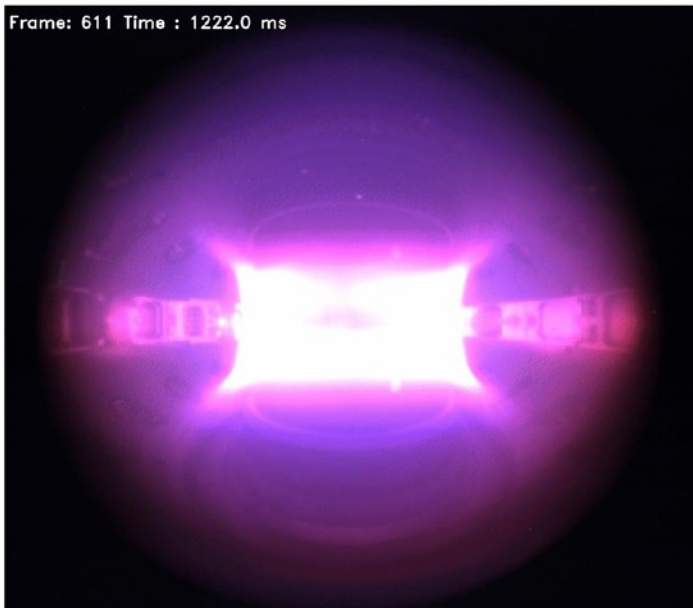
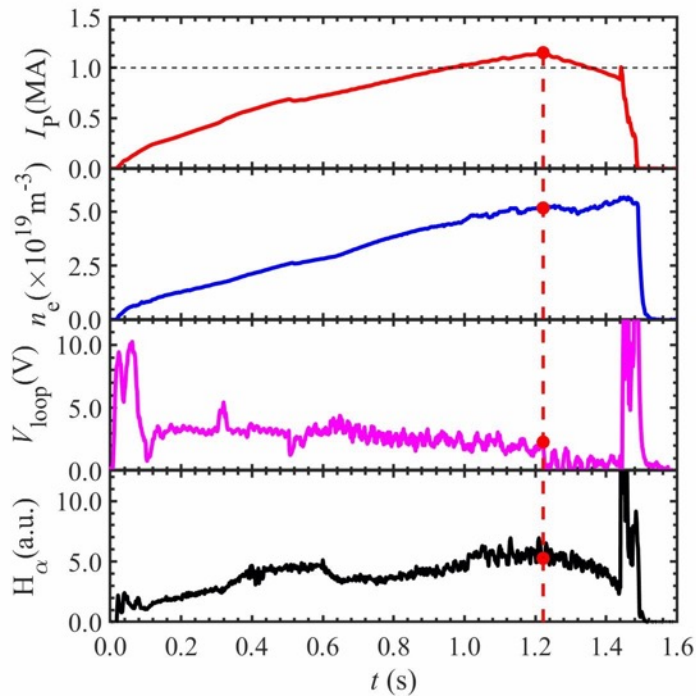
First plasma:  $>100$  kA,  $>100$  ms

Reported by CCTV News on Dec. 4th, 2020



"Top 10 News of Scientific and Technological Progress in China in 2020"

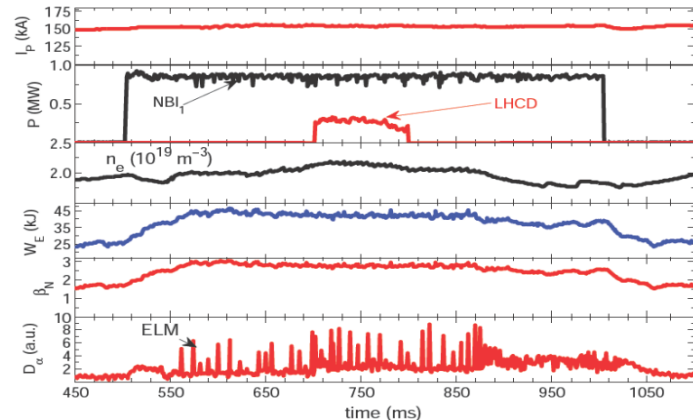
# HL-2M has operated with $I_p > 1$ MA, $n_e > 5 \times 10^{19} \text{ m}^{-3}$ on Oct. 19<sup>th</sup>, 2022



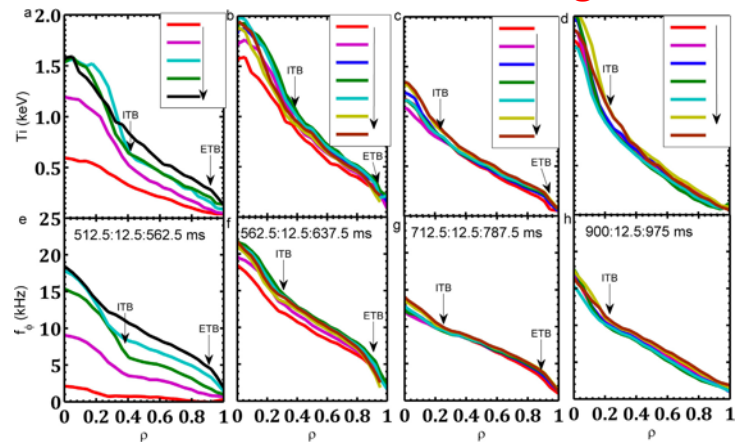
- Inner-Wall Limited Configuration
- $I_{p,max} = 1.15 \text{ MA}$
- $R = 1.8 \text{ m}$
- $a = 0.69 \text{ m}$
- $\kappa = 1.25$
- $\delta = 0.14$

# High $\beta_N$ regimes based on double transport barriers on HL-2A

- **High  $\beta_N$  regime accessed by NBI at low Bt  $\sim 1.3T$** 
  - Sustained  $\beta_N > 2$ , for  $t \sim 15\tau_E$
  - Transient:  $\beta_N > 3$  @  $f_G \sim 0.6$ ,  $H_{98} \sim 1.5$ ,  $q_{95} \sim 4.0$
- **Hybrid scenario with H-mode edge**
  - ITB with high  $f_\phi$  and central weak shear
  - the highest  $\beta_N$  achieved at strong DTBs
- **Interesting studies in High  $\beta_N$  H-mode plasmas:**
  - MHD instabilities and Alfvénic modes (TAEs and BAEs/EPMs, etc)
  - ITB dynamics by internal kink mode and impurity injection
  - Non-linear mode-coupling



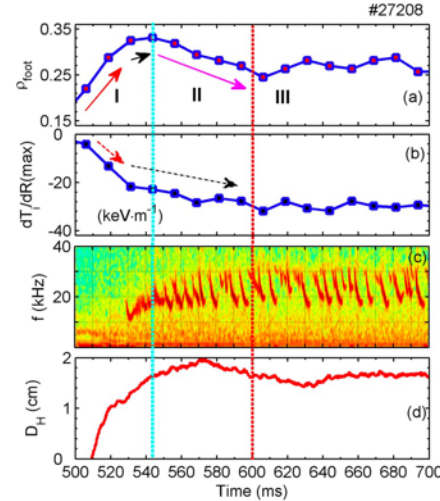
**Minor radius ITB + Strong ETB**



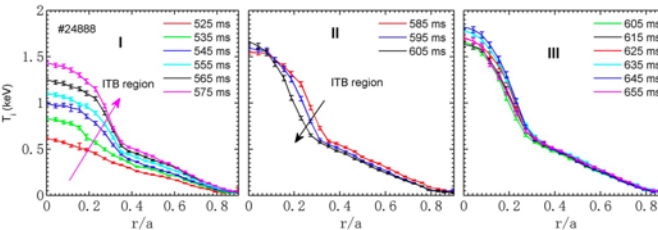
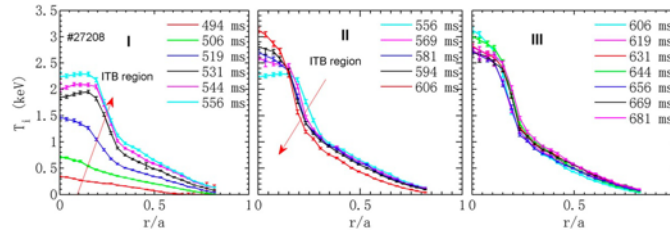
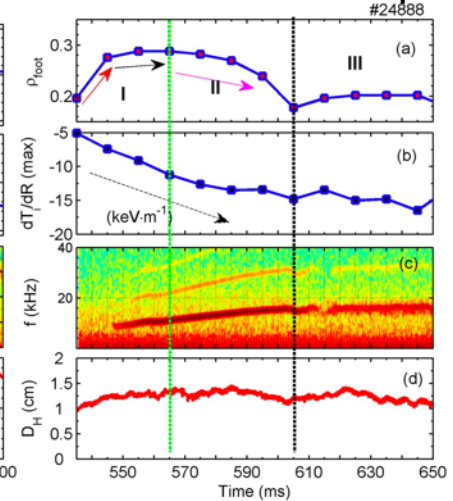
# ITB dynamics controlled by internal kink mode

- ITB formed before IKM
- ITB evolved in three phases
  - I: ITB build-up with increasing radial position and strength
  - II: shrinks inward as appearance of IKM
  - III: stationary with saturated IKM
- IKM clamps the ITB foot @  $q \sim 1$  (HB, magnetic flux pumping?)
- ITBs at larger radius can exist in quiet core plasma, but not be stationary

**FB ITBs @ lower  $n_e$  and higher  $P_{NB}$**

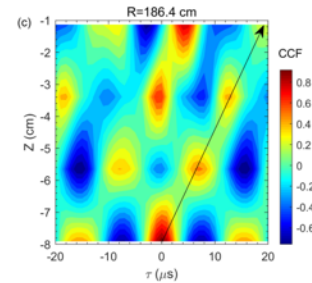
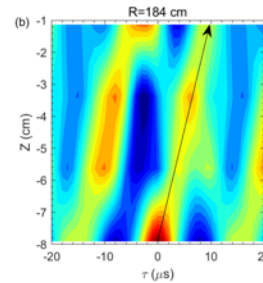
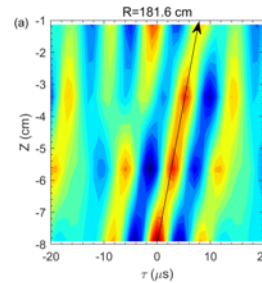
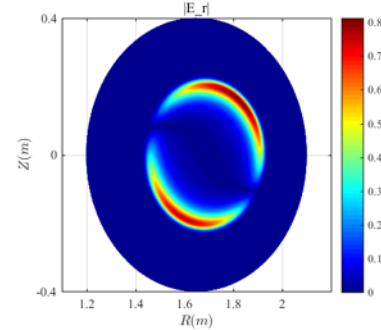
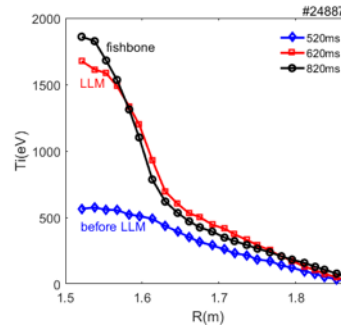
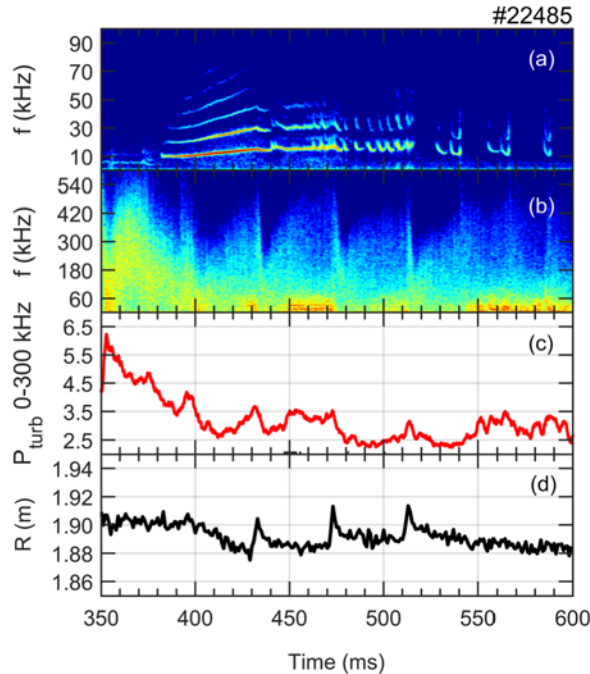


**LLM ITBs @ higher  $n_e$  and  $I_p$**



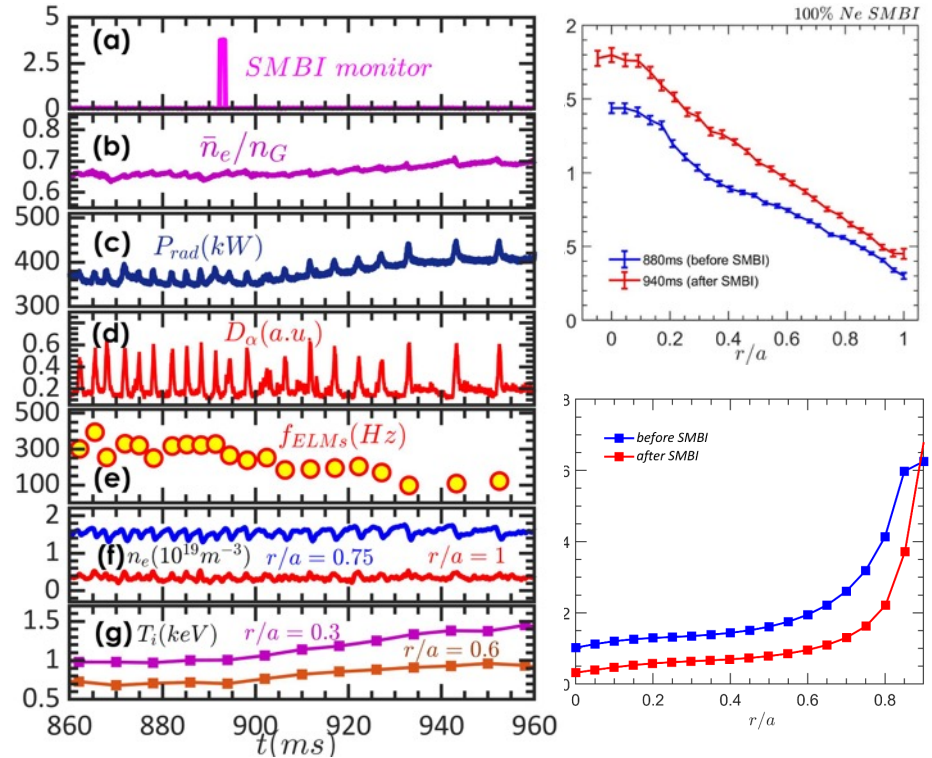
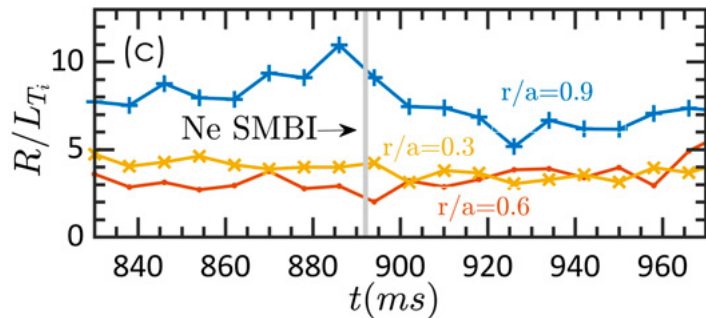
# Role of fishbone activity in the formation of ITB

Experimental evidences suggest that the fishbone activities can induce a poloidal flow, which is beneficial for the suppression of turbulence in the plasma core region.



# Enhancement of $T_i$ in H-mode by impurity seeding

- Reduce the ELM frequency and improve the plasma confinement
- Decreased ITG turbulent transport
- Broader pedestal profile prolonged inter-ELM periods
- Unchanged the core ion stiffness





# ELM control with impurity seeding

## ● Plausible mechanism :

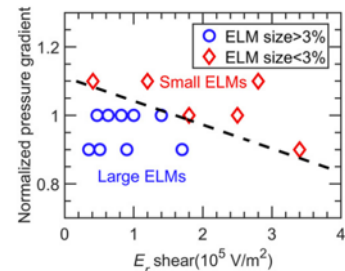
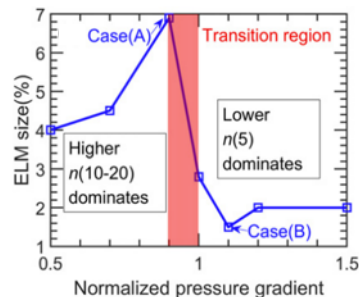
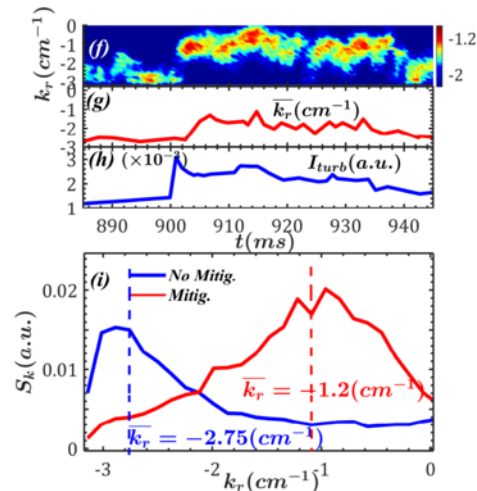
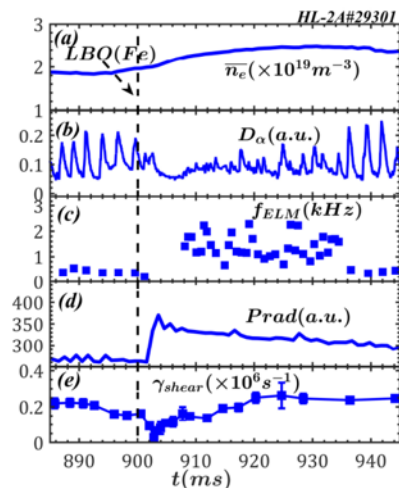
- Edge velocity shear decrease
- Turbulence radial spectral shift
- Turbulence enhancement
- ELM mitigation

## ● Simulation:

ELM size can be determined by the changes in pedestal pressure and  $E_r$  shear due to the impurity seeding

G.L.Xiao et al., Nucl. Fusion 61:116011 (2021)

Y. R. Zhu et al., Nucl. Fusion 62:076011 (2022)



# Avalanche Transport Events Triggered by Nonlinear Mode Couplings

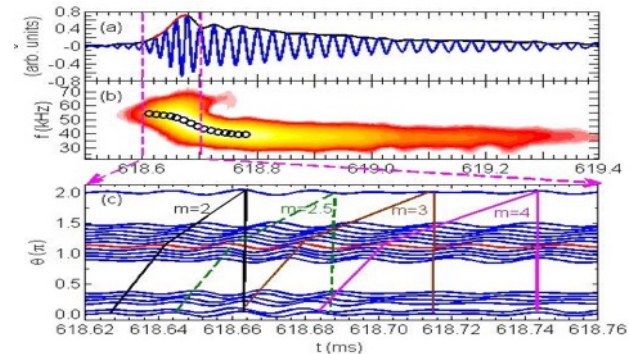
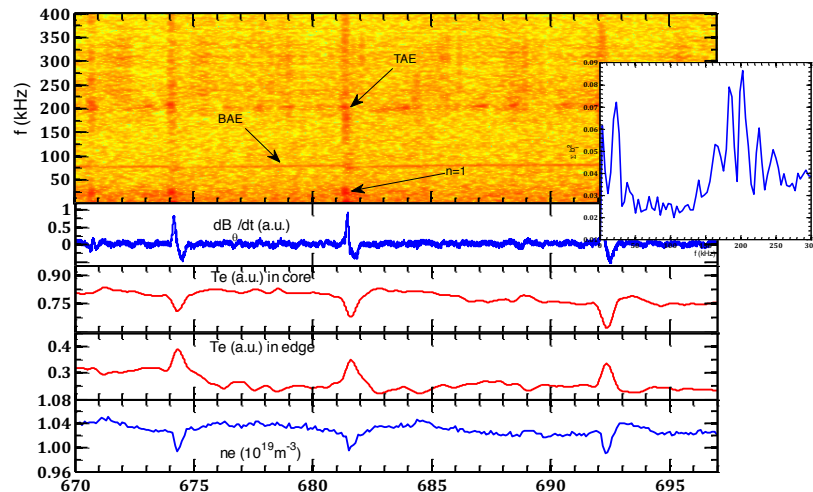
- The nonlinear mode coupling between the core-localized TAEs and  $m/n = 2/1$  fishbone generate multiple modes, cause resonance overlap in real and phase spaces, trigger the avalanche event, enhance electron heat transport

W. Chen, et al., *Nucl. Fusion*, 60 094003 (2020)

- In a strong EPM burst (in another shot), mode radial propagation ( $m=2 \rightarrow 4$ ) causes significant EP transport (EPM convective amplification)

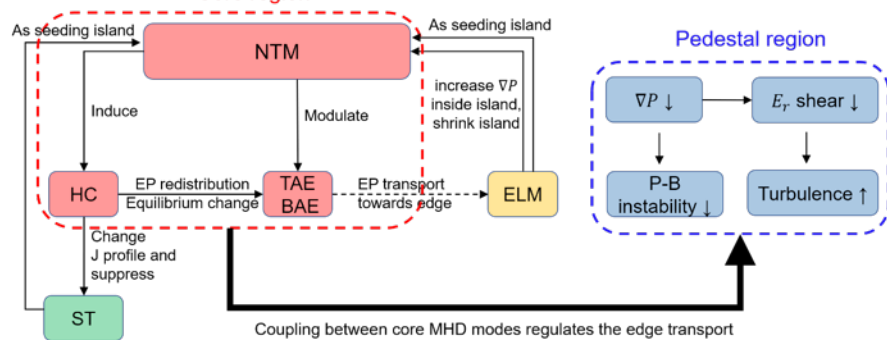
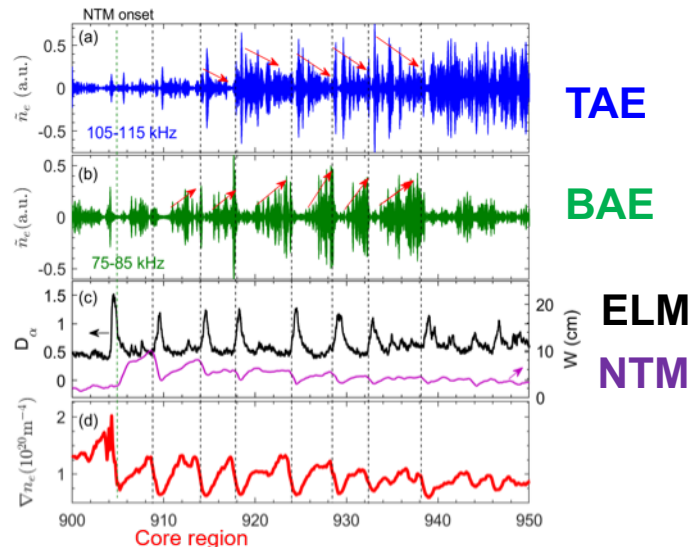
L.M. Yu, et al., *Europhys. Lett.* 138, 54002 (2022)

- The non-linear couplings among different scales are crucial for the core-edge integration



# Interaction among core MHD modes and its impact on pedestal transport

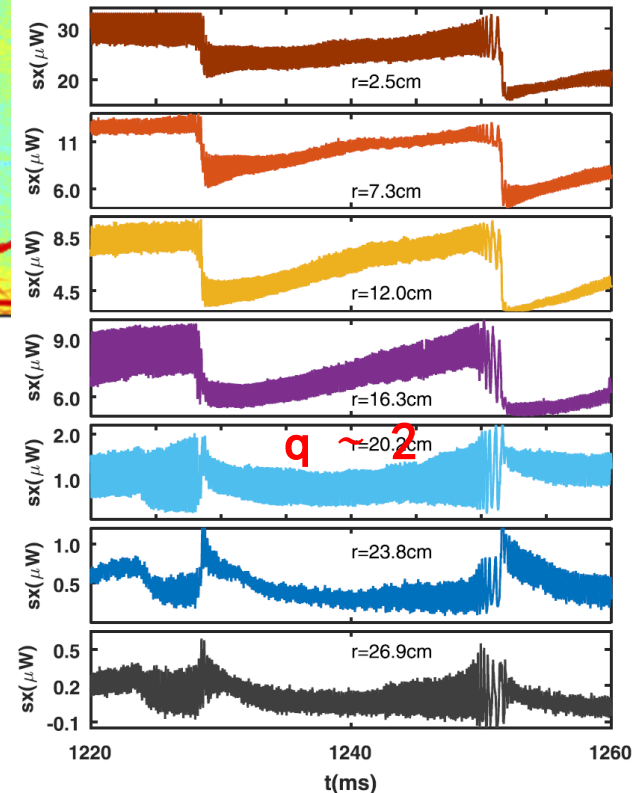
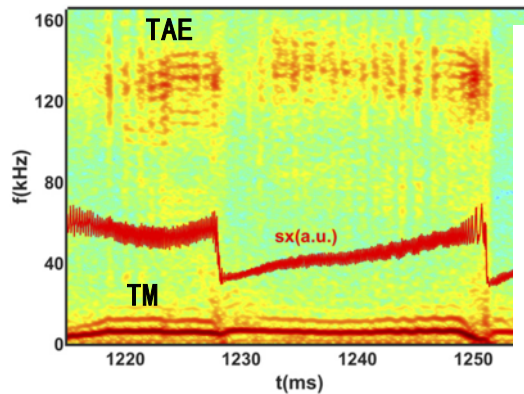
- NTM(3/2 or 2/1), TAE(@q ~ 3) /BAE (@q ~ 2) and ELMs co-exist in high  $\beta_N$  H-mode plasmas
- They closely interacted with each other
  - NTM modulate TAE and BAE
  - ELM crash causes a rapid drop of  $W_{\text{NTM}}$
  - accompany a saturated 1/1 Helical core (magnetic flux pumping in hybrid?)
  - enhance the fast ion transport
- Such interaction causes increase of pedestal turbulent transport.
- Core-edge interaction!



# Off-axis sawtooth induced by nonlinear mode coupling

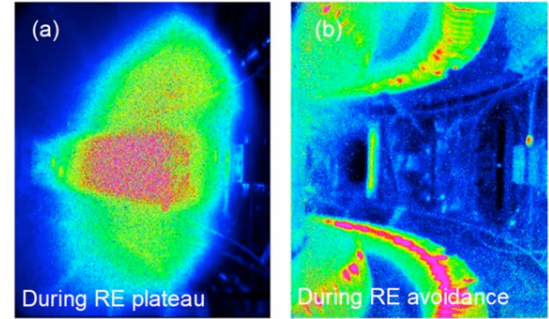
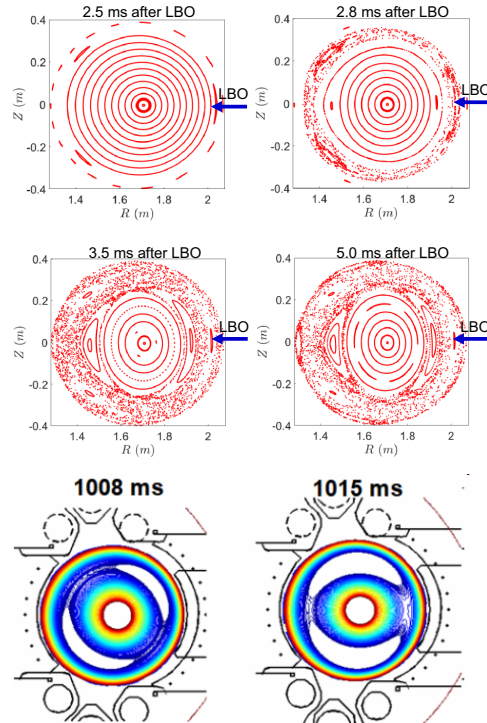
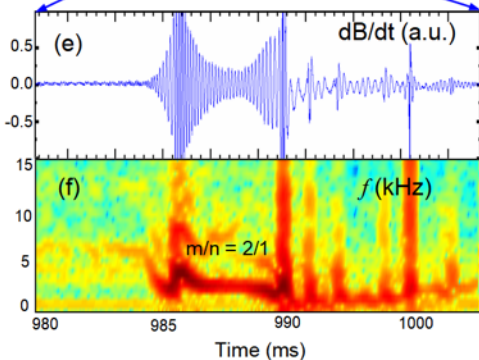
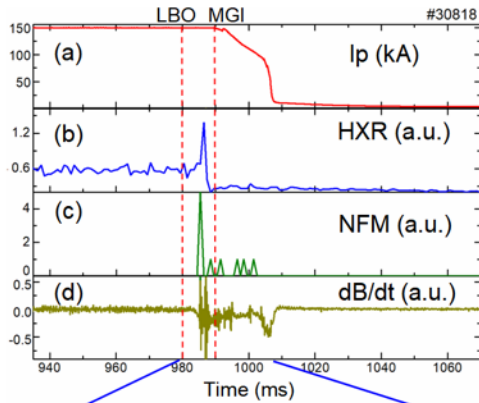
- Off-axis sawtooth @  $q \sim 2$  induced by nonlinear mode coupling between TAE and tearing mode

nonlinear coupling enhances fast ion outward transport and leads to loss of toroidal torque,  
→ results in rotation braking or mode locking,  
→ saturated tearing mode leads to subsequent collapse



# Avoidance of RE generation during disruptions

RE generation during disruptions has been successfully avoided for the first time by the LBO-seeded impurity in the HL-2A tokamak.



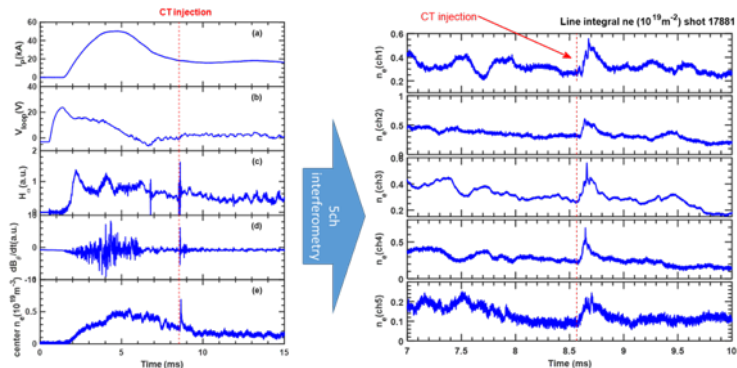
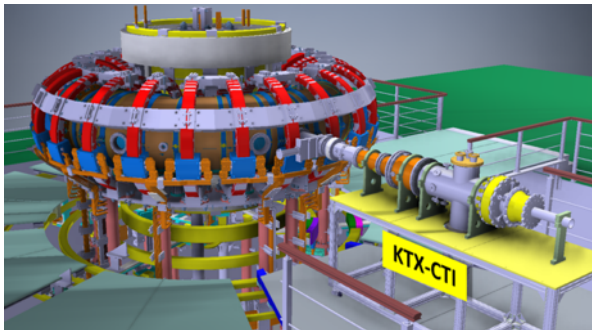
- A strong  $m/n=2/1$  mode is excited by LBO-seeded impurity.
- The magnetic field outside  $q = 2$  is completely randomized, which leads to the fast losses of REs.



# **Fusion Research in Universities**

# Keda Torus eXperiment in USTC

## First compact torus injection experiment



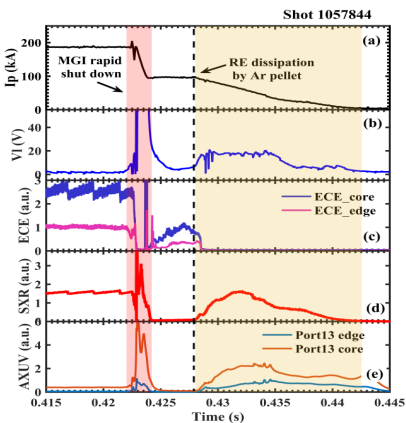
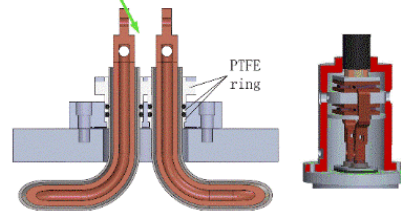
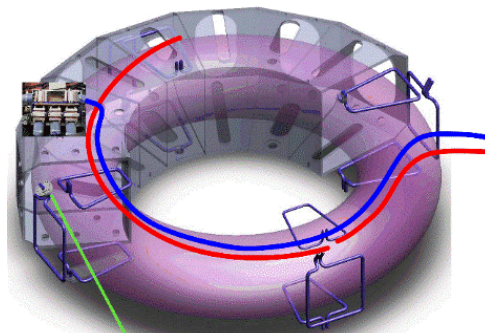
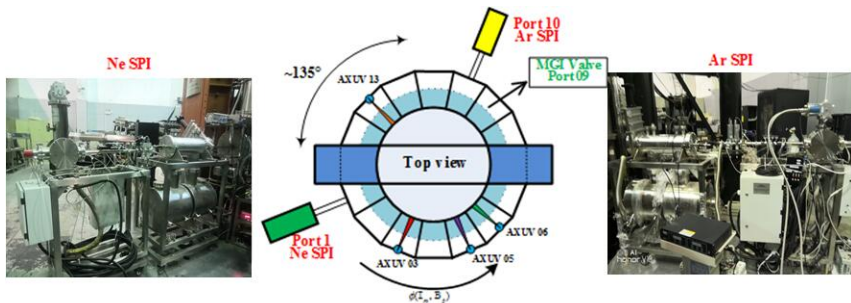
Parameters	Values
Formation bank voltage	20kV
Acceleration bank voltage	20kV
Solenoid bank voltage	3kV, $\Psi_{bias} = 6\text{mWb}$
Gas puffing bank voltage	2000V
Outer diameter ( $D_{ct}$ ) at the exit	8.5cm
<b>Maximum Velocity (<math>V_{ct}</math>)</b>	<b>300km/s</b>
Mass ( $m_{ct}$ )	0.1mg
Number of particles	$1 \times 10^{20}$
% fueling (of KTX inventory)	5~50%
Length of the injector	3m
Tangential injection angle	$25^\circ$

- **KTX Upgrade** allow 1MA operation:
- ✓ KTX machine are now being upgrade to its full capacity as designed, including Power supply, feedback control and diagnostic systems
- ✓ Aiming at next step research on 3D physics in MCF plasmas and general physics between Tokamak and RFP configuration
- ✓ Testing new concepts and technologies of diagnostics

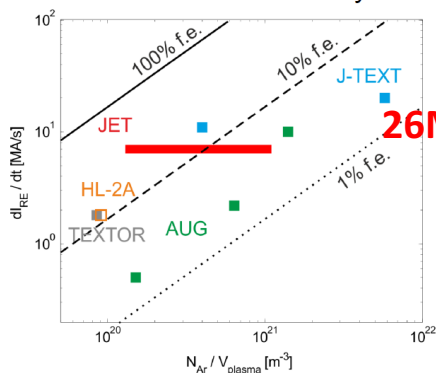


# J-TEXT in HUST

## Equipped dual SPI in support of ITPA task force

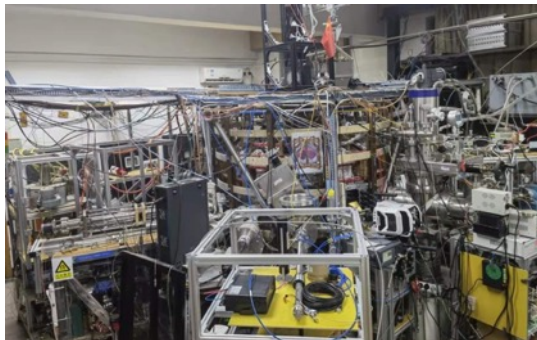


RE current decay rate as function of Ar density



Rotating ( 6kHz ) magnetic perturbation

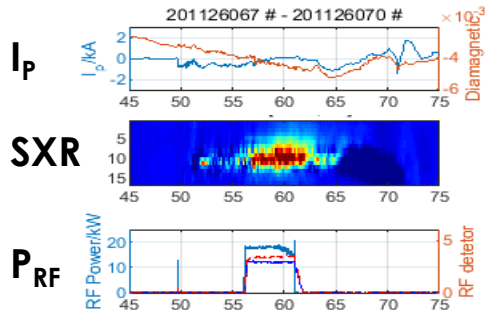




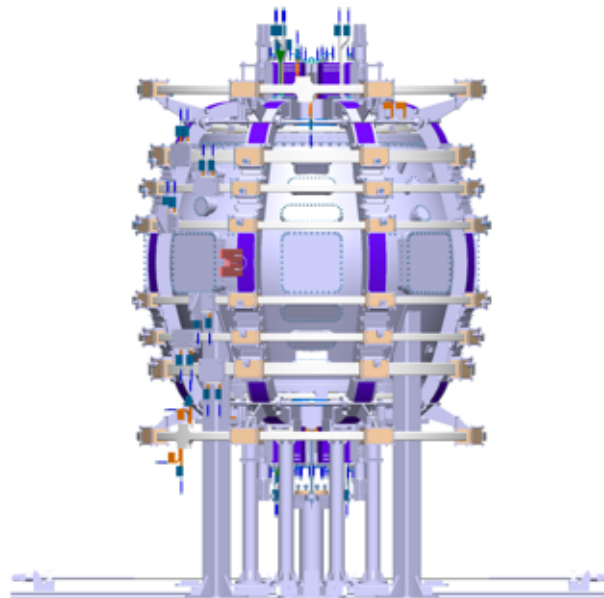
## A merging experiment in SUNIST-II

### Main Parameters:

- ✓  $R/a \sim 0.5\text{m}/0.3\text{m}$ ;  $I_p \sim 200\text{-}400\text{kA}$ ;  $B_{t0} \sim 0.4\text{-}1\text{T}$ ;  $\kappa \sim 1.6$



- Successful EBW heating
- density fluctuations of TAEs
- equilibrium reconstruction





# Very active research on theory & simulation in universities

**Most projects on energetic particle physics focused in theory/modeling/diagnostics and allocated to Universities in recent 5 years, collaboration with two major facilities are requested**

- 氘氚聚变等离子体中alpha粒子过程对等离子体约束性能影响的理论模拟研究 (Transport, IP CAS)
- 面向聚变堆高性能等离子体中快粒子物理实验研究(Experiments, SWIP)
- 氘氚聚变等离子体中磁流体过程的理论和模拟研究(EPM, UZJ)
- 基于非线性回旋动力学的氘氚聚变等离子体约束改善理论和模拟研究(NI-Gyro-kinetic, USTC)
- 聚变堆等离子体加料、离子加热模块研发(Fueling and heating, UDL)
- 提升聚变堆等离子体密度参数的关键问题研究( density limit, UZJ)
- 燃烧等离子体中快粒子对芯部约束影响的理论模拟研究(RF-EP, USTC)
- 高分辨热相干散射、安全因子、中子探测诊断技术研究( Diagnostics, USTC)
- 聚变堆燃料粒子合成诊断技术和方法研究(Diagnostics, PKU)
- 聚变堆条件下磁流体稳定性和高能量粒子若干挑战性问题的研究 (MHD&EP, Talent)
- 聚变堆条件下约束与输运的若干关键问题研究(T&C, Talent)



# Personal perspective on BEST

- **Milestone “Break-Even” under Steady-state**
  - Steady-state scenario exist with sufficient EPs?
  - Hybrid scenario with EPs for long pulse operation
  - EP physics applicable to scenario development
- **At least a proven approach to provide TBR>1**
  - Extrapolatable to DEMO
  - Particle balance issue (fueling, burning, retention, recycling, impurities,...)
- **Constraints on cost and Tritium/neutron budget**
  - Prediction-first
  - Baseline rely on proven physics
  - Exploration from aggressive AT physics

ITER delayed; CFETR difficult in near future

