

Electromagnetic modelling of DTT scenarios and control

A. Castaldo



Italian National Agency for New Technologies,
Energy and Sustainable Economic Development

Outline

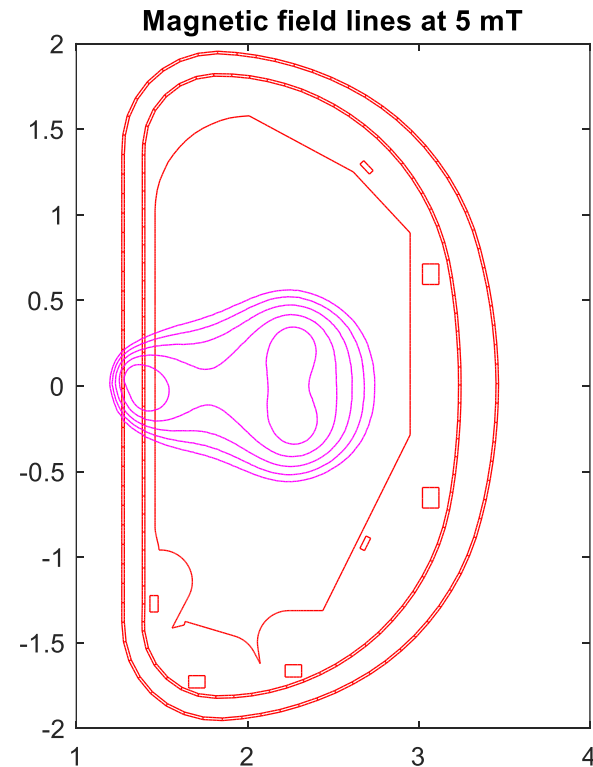
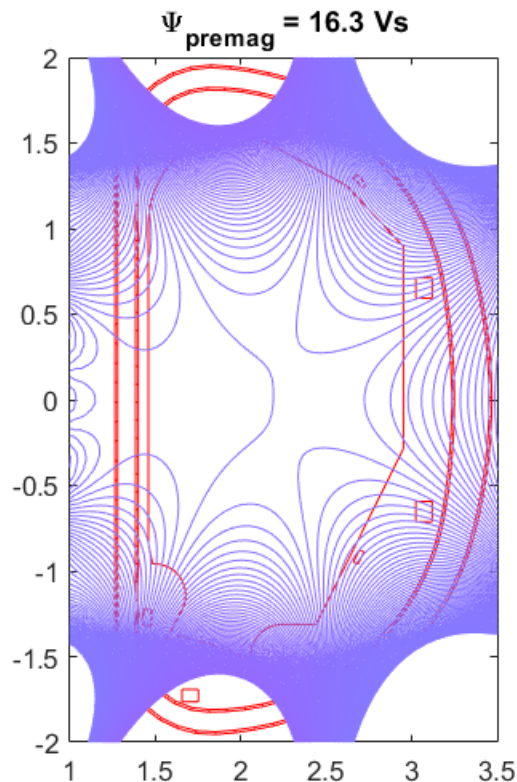
- Education and training
 - Activities carried out in 2023:
 - Definition of plasma electromagnetic scenarios and weak coupling with transport code ASTRA for DTT (scenarios A, C and E);
 - Disruption simulations for DTT;
 - Sweeping control;
 - Creation of IMAS structures for DTT;
 - Foreseen activities for 2024;
 - Definition of B-D scenarios and XD and NT for DTT;
 - Proposal for collaboration with WEST team;
 - Kinetic control for DTT;
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Education and training

- B.Sc and M.Sc in Management Engineering (Energy address) from University of Naples 'Parthenope';
 - PhD student (XXXI° cycle) of Joint European Doctorate in Fusion Science and Engineering (University of Padova, University of Naples 'Federico II', ENEA, Max Planck Institute for plasma physics);
 - Employee of CREATE Consortium and L.T. Calcoli;
 - Researcher at ENEA (TD) research center in Frascati (from 01/06/2022 to 01/06/2023);
 - Currently researcher at ENEA (TI) research center in Frascati;
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DTT plasma scenarios (I)

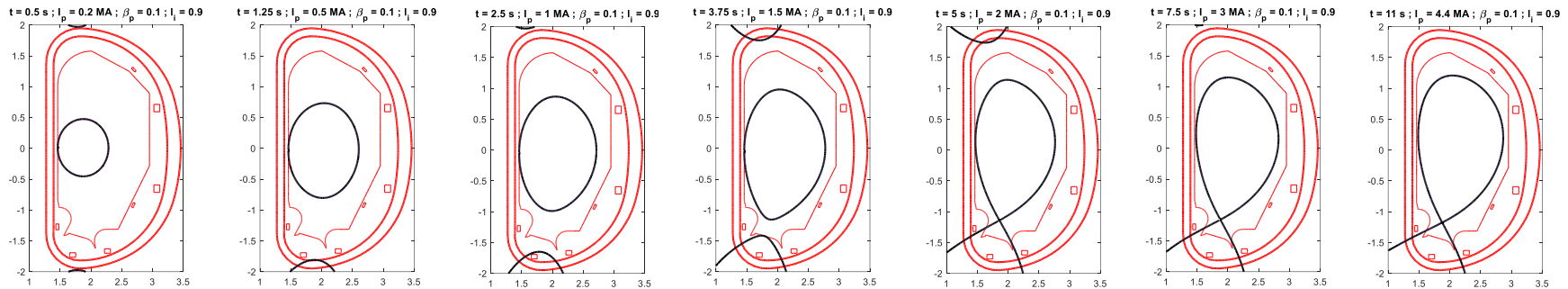
- In 2023 plasma scenarios A, C and E have been released and uploaded on Alfresco [Alfresco » Document Library \(enea.it\)](https://alfresco.enea.it/);
- The starting point for the definition of a scenario is the evaluation of the premagnetization phase;



DTT plasma scenarios (II)

A preliminary definition of the ramp-up is realized starting from:

- Simplified sequence of the plasma shapes



- Simplified plasma current profiles (Bell shape profiles)

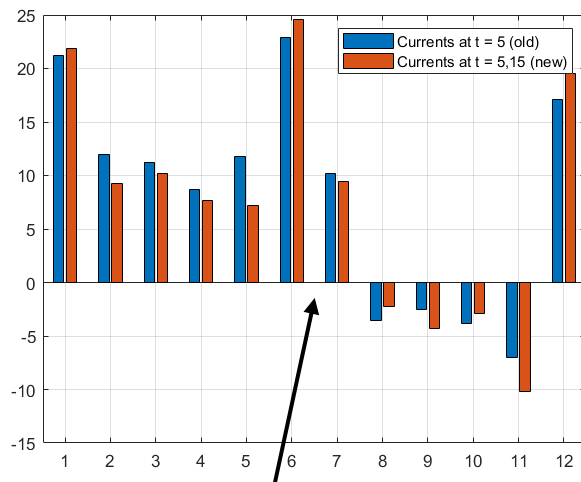
- Total plasma current I_p
- Poloidal beta $\beta_{pol} = W_p / W_m$
- Internal inductance $l_i = 4W_m / (\mu_0 R_0 I_p^2)$

- Simplified assumption of the plasma boundary flux consumption (Ejima approach): $\Psi(t) = 0,5\mu_0 R_0 l_i I_p + C_{Ejima} \mu_0 R_0 I_p$

DTT plasma scenarios (III)

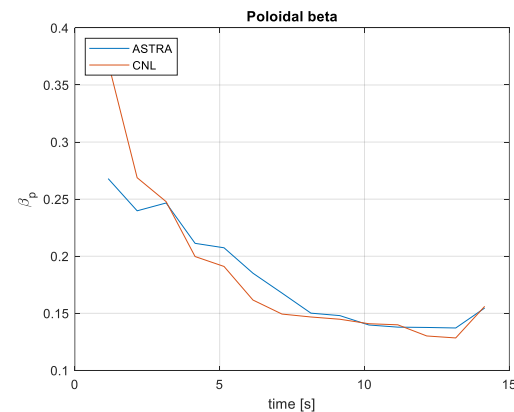
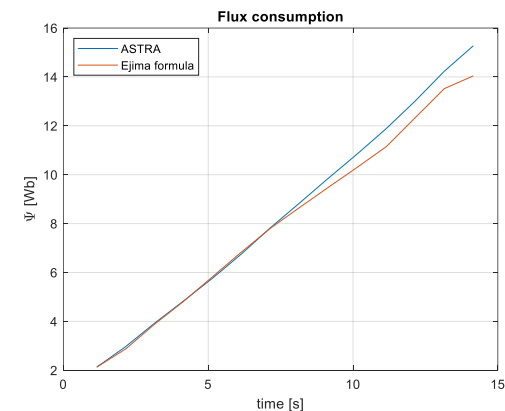
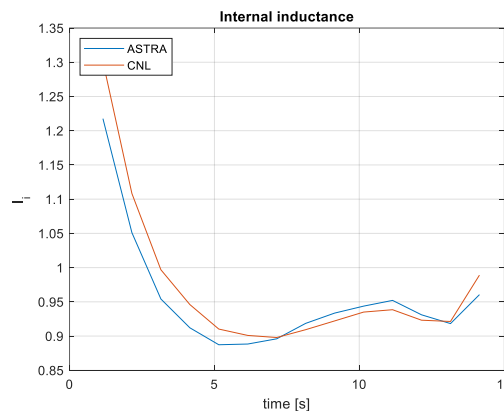
- In order to validate the proposed ramp-up, transport simulations performed with JINTRAC have been carried out;
- Since JINTRAC offers as outputs poloidal current function (f), pressure (p) and their derivatives, these quantities can be used as inputs of the e.m. code

CREATE-NL+;



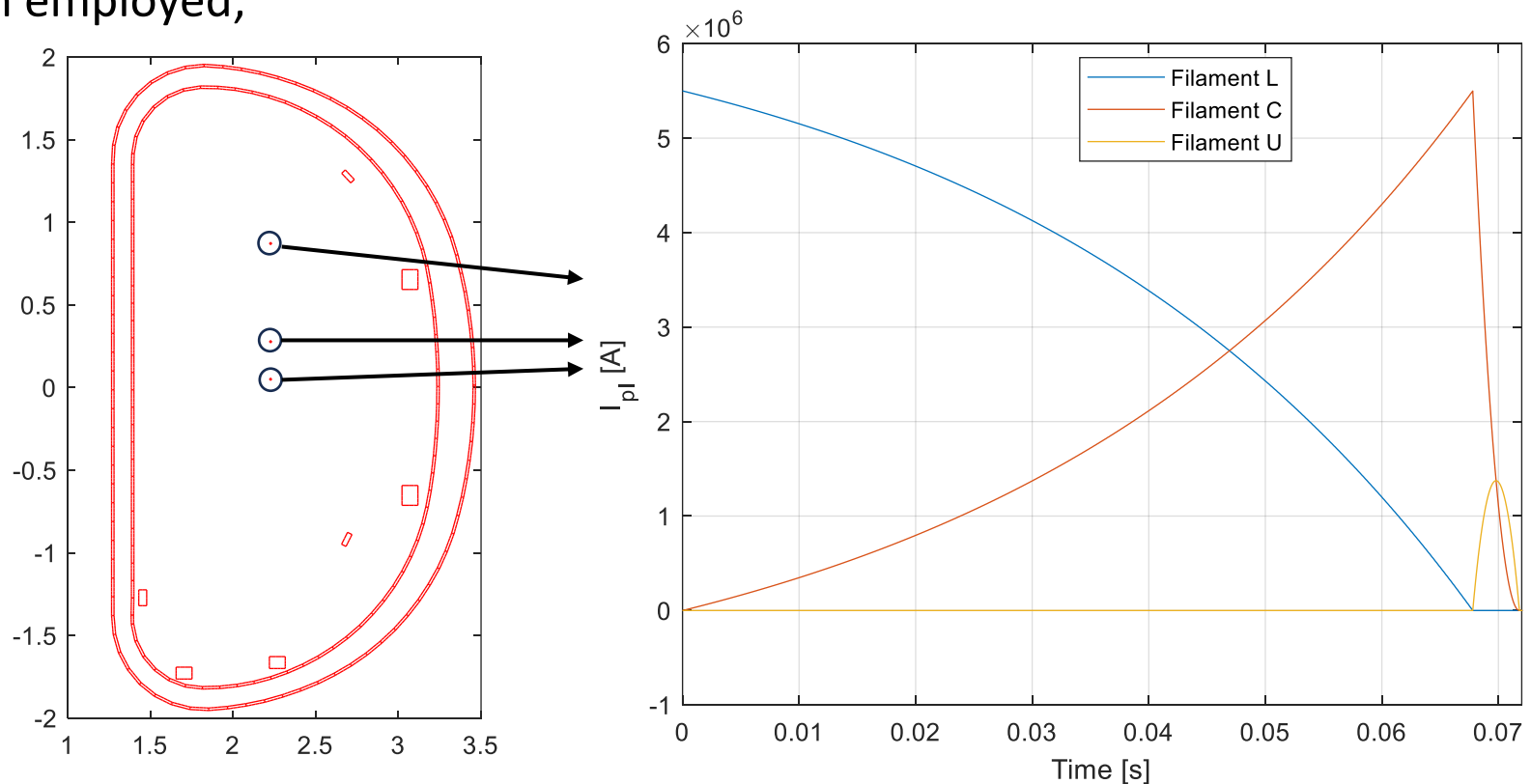
A tuning of CS/PF currents turns out to be necessary in order to fit plasma shape

N. Bonanomi et al, 'Time-dependent full-radius DTT scenarios prediction with reduced transport models', TBD



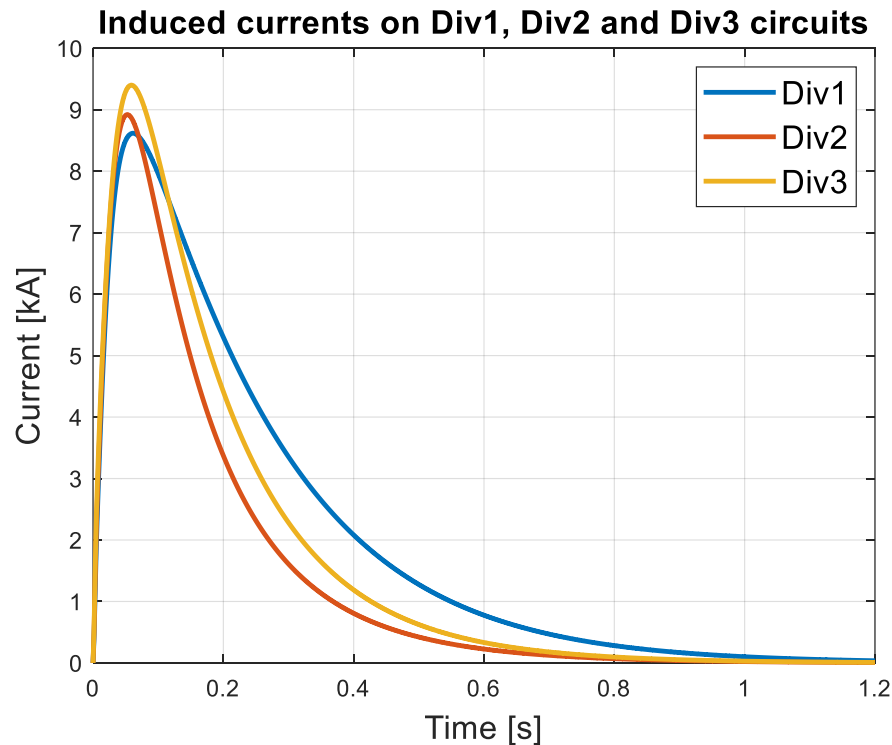
Disruption analysis for DTT in-vessel coils (I)

- Disruptive events can be very dangerous for the components of a tokamak machine, in particular for the elements closer to plasma column such as in-vessel coils;
- In order to simulate a disruption in DTT a simplified 3 filaments model has been employed;



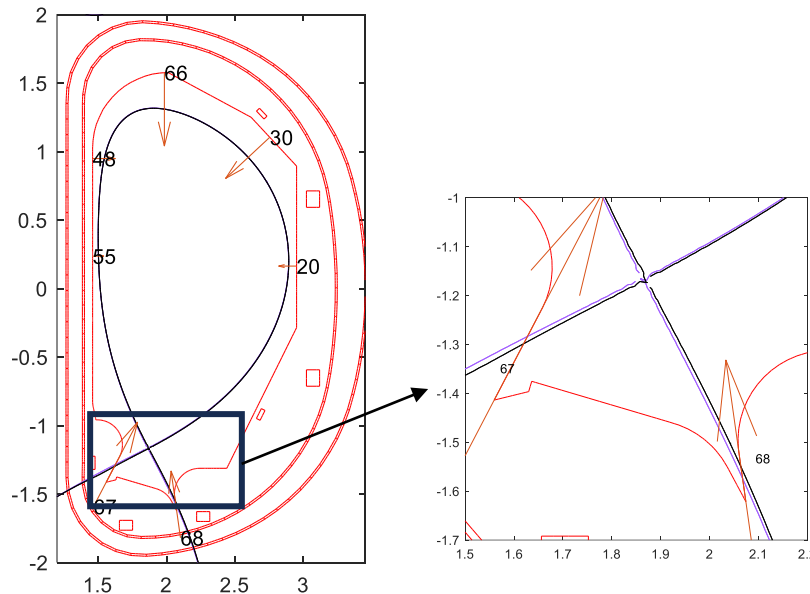
Disruption analysis for DTT in-vessel coils (II)

- In order to protect in-vessel coils during disruptions, additional inductances and resistances can be placed to reduce the current peaks during disruptions;
- The results obtained by linear simulation have been validated recently by non-linear simulations performed with CREATE-NL+ code;



Sweeping control for DTT (I)

- Since one of the main objective of DTT is to study different solutions for the power exhaust problem, a set of dedicated coils has been placed in the divertor region;
- These coils can be used for different purposes such as: assisting X-point formation, local control of flux/magnetic field in the divertor region, implementation of advanced control techniques such as strike-point sweeping;



Shape Gaps: 20, 30, 48, 55, 66

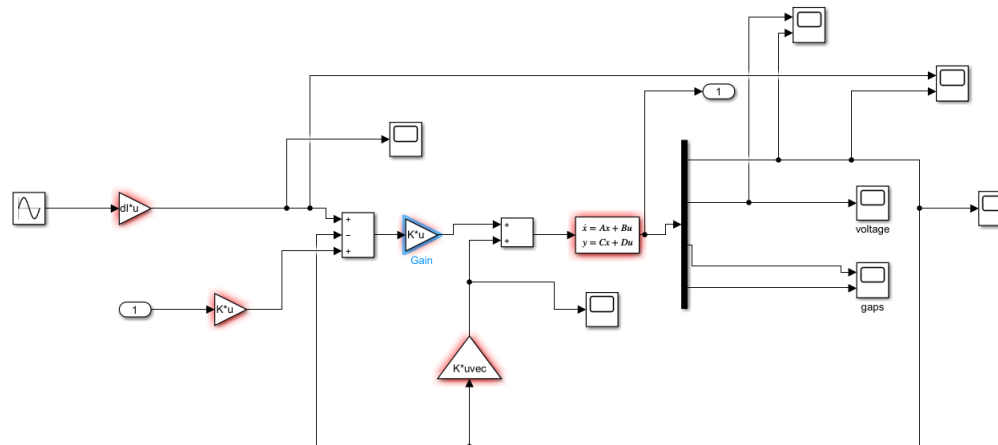
Sweeping Gaps: 67,68

ΔI_{sweep}^{eq} able to decouple
sweeping of plasma legs and
movement of plasma shape

Sweeping control for DTT (II)

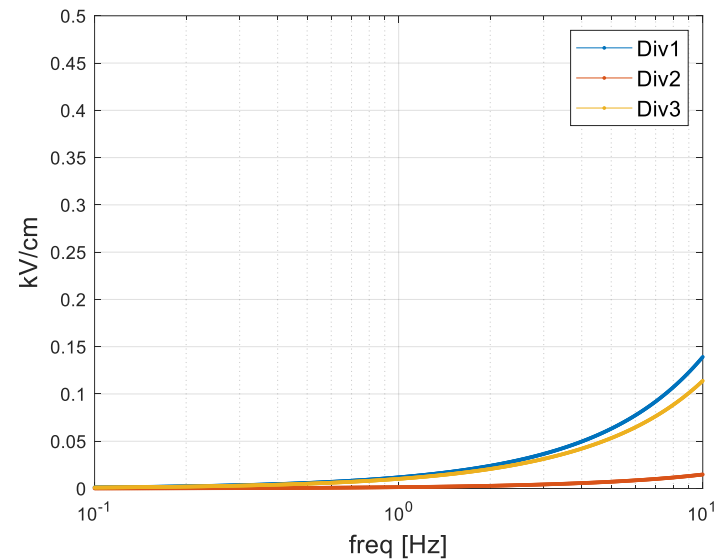
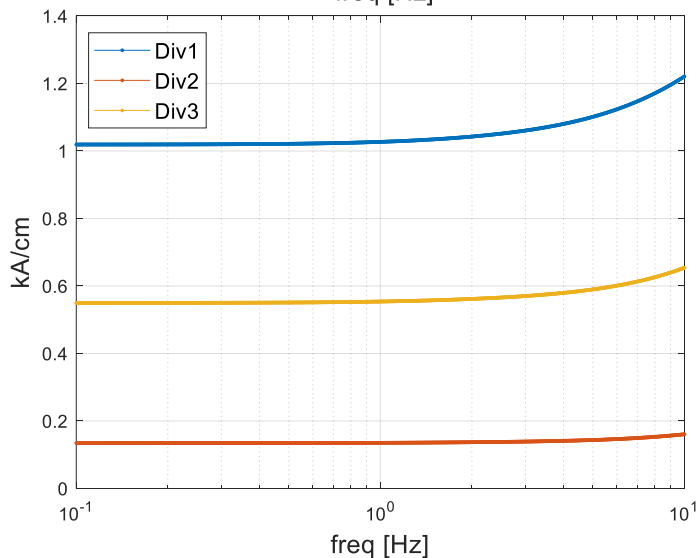
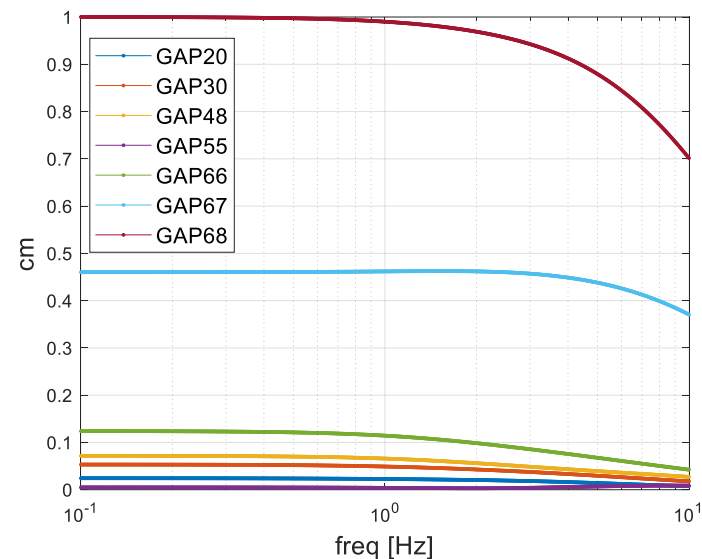
- Once computed ΔI_{sweep}^{eq} in order to implement legs sweeping the current control law to impose is $I_{sweep}(t) = \Delta I_{sweep}^{eq} \sin(\bar{\omega}t)$ where $\bar{\omega}$ is the sweeping frequency;
- In order to obtain the desired current dynamics since divertor coils are voltage driven the following voltage control law can be imposed (*):

$$V_a(t) = \frac{L}{\tau} \Delta I_{sweep}^{eq} \tilde{V}(t) - \frac{L}{\tau} I_a(t) + R I_a(t)$$



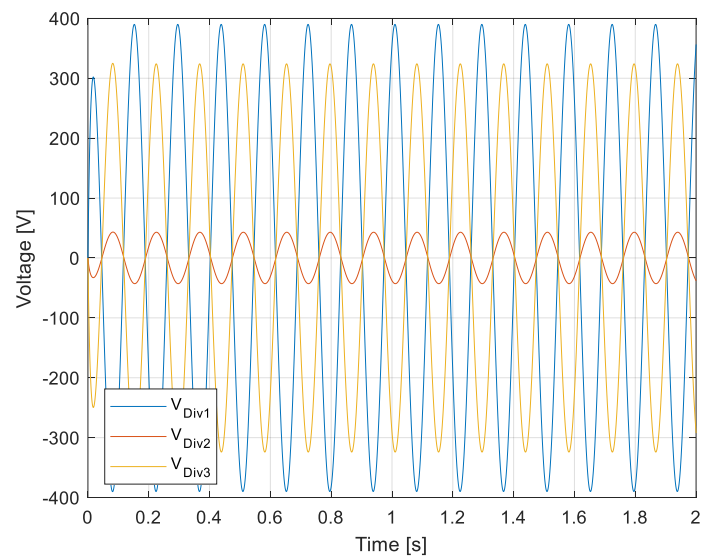
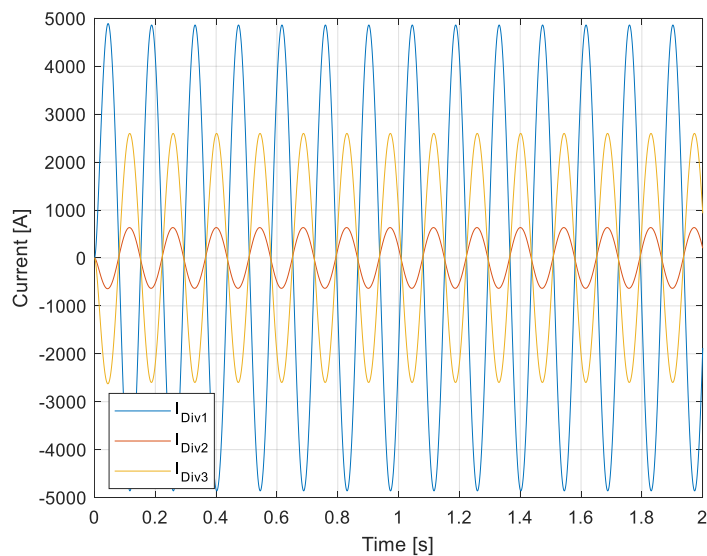
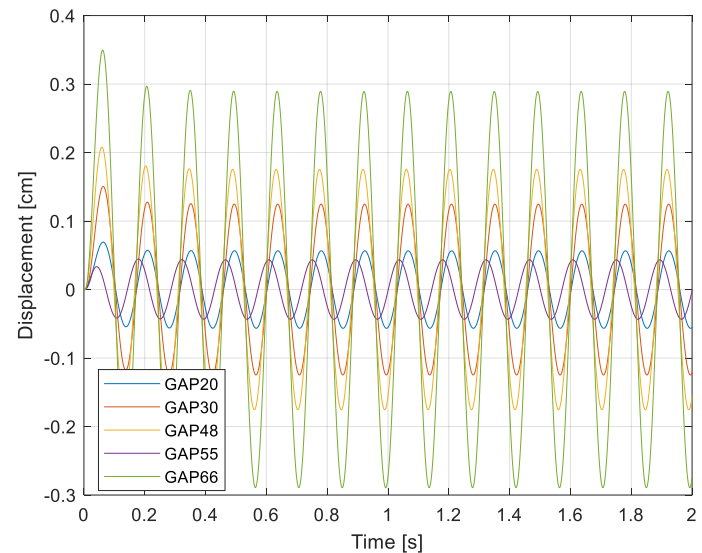
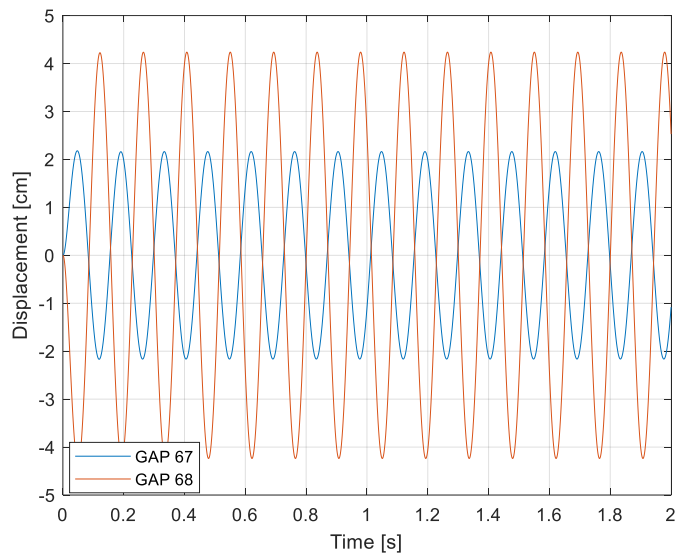
(*) – R. Ambrosino et al, 'Sweeping control performance on DEMO', Fusion Engineering and Design, Volume 171, October 2021

Sweeping control for DTT (III)



- Good decoupling between sweeping gaps and shape gaps in the frequency range [1 – 10] Hz;
- According to voltage limit a sweeping amplitude of ± 4 cm at 7 Hz can be imposed on the outer target and ± 2 cm on inner target;

Sweeping control for DTT (IV)



IMAS for DTT

- The Integrated Modelling & Analysis Suite (IMAS) is the collection of software that will be used for all physics modelling and analysis. It will represent the standard for ITER;
- The outputs from 2D FEM CREATE-NL+ code can be used (eventually postprocessed) in order to fill in a sub-set of IMAS fields;

IMAS Data Model (3.34.0)

		Heating systems	Diagnostics
amns_data	disruption	iron_core	reflectometer_profile
barometry	distribution_sources	langmuir_probes	refractometer
bolometer	distributions	lh_antennas	sawteeth
bremssstrahlung_visible	divertors	magnetics	soft_x_rays
calorimetry	ec_launchers	mhd	spectrometer_mass
camera_ir	ece	mhd_linear	spectrometer_uv
camera_visible	edge_profiles	mse	spectrometer_visible
charge_exchange	edge_sources	nbi	spectrometer_x_ray_crystal
coils_non_axisymmetric	edge_transport	neutron_diagnostic	summary
controllers	em_coupling	ntms	temporary
core_instant_changes	equilibrium	pellets	thomson_scattering
core_profiles	gas_injection	pf_active	tf
core_sources	gas_pumping	pf_passive	transport_solver_numerics
core_transport	gyrokinetics	polarimeter	turbulence
cyrostat	hard_x_rays	pulse_schedule	wall
dataset_description	ic_antennas	radiation	waves
dataset_fair	interferometer	real_time_data	workflow

2024 foreseen activities

- For 2024 the activities foreseen are the following:
 - Revision of A, C and E scenarios on the new CS version (v16) with two layers and coupling with ASTRA for all the scenarios phases;
 - Definition of B and D scenarios on the latest version of the machine design;
 - Study of alternative configurations (XD and SN-NT) with the definition of proper electromagnetic scenarios compatible with machine constraints;
 - Further disruption analysis to be defined in order to study the behavior of the 2 layer CS;
 - Definition of a complete DTT scenario on IMAS workflow;
 - Start of the activities on kinetic control (definition of the model, controllers tuning, identification of the available diagnostics);
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Proposal for WEST activities

- Electromagnetic analysis of the WEST tokamak:
 - Definition of the 2D mesh, development of linearized models of the device;
 - Testing of the E.M. models developed by simulation of dry-runs;
 - Validation of models with plasma developed by comparing plasma shape and physical quantities obtained with reconstruction codes employed on WEST;
 - Analysis and optimization of ramp-up, flat-top and ramp-down phases;
 - All the analysis will be carried out using 2D FEM solver CREATE-NL+ code and CREATE-L code for linearized plasma response;
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